

NRR-PMDAPEm Resource

From: Hon, Andrew
Sent: Wednesday, March 30, 2016 4:56 PM
To: Caves, John (John.Caves@duke-energy.com); McDaniel, Sarah
Subject: Shearon Harris Nuclear Power Plant – Request for Additional Information related to LAR - Main Steam Safety Valve Lift Setting Tolerance Change (TAC NO. MF7195)

In a letter dated December 17, 2015, (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML15362A169), Duke Energy Progress (DEP, the licensee) requested the subject amendment to Renewed Operating No. NPF-63.

The U.S. Nuclear Regulatory Commission (NRC) staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is listed below:

1. Table 2 of the LAR provides a summary of the FSAR chapter 15 event dispositions related to the increased setpoint tolerance change. The results presented in this table are based on Areva Document No.: 51-9237347-002, "Harris PSV and MSSV Tolerance Increase Disposition of Non-LOCA Events." This document, which was reviewed during the March 2016 audit, contains the following statement for numerous events:

"This disposition will require further verification (See Assumption #1 in Section 4.0) by Duke to confirm that the revised Turbine Trip analysis continues to bound the (VARIOUS EVENTS)."

Please confirm that DEP has performed the verification that the revised turbine trip analysis continues to bound the individual events listed in Table 2 of the LAR.

2. Please explain the reason for the change in the Pressure Safety Valve setpoint uncertainty in the Turbine Trip analysis and its impact on the calculation results.
3. Please explain how the "Dispositions" in the Table 2 of the LAR (Summary of FSAR Chapter 15 Event Dispositions) are developed.

RAI basis: NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition," (SRP) Section 5.2.2, Revision 3, "Overpressure Protection,".

The proposed questions were discussed with your staff during an audit on March 2, 2016. Your staff also confirmed on March 29, 2016, that this request for additional information (RAI) was understood and did not include proprietary or security-related information, and agreed to provide a response in 30 day from this request. The NRC staff considers that timely responses to RAIs help to ensure sufficient time is available for staff review and contribute toward the NRC's goal of efficient and effective use of staff resources. Please note that if you do not respond to this request by the agreed-upon date or provide an acceptable alternate date, we may deny your application for amendment under the provisions of Title 10 of the *Code of Federal Regulations*, Section 2.108. If circumstances result in the need to revise the agreed upon response date, please contact me at (301) 415-8480 or via e-mail Andrew.Hon@nrc.gov.

Andy Hon, PE

Project Manager (Brunswick Nuclear Plant 1 & 2, Sequoyah Nuclear Plant 1 & 2)

Plant Licensing Branch II-2

Division of Operating Reactor Licensing

Office of Nuclear Reactor Regulation

301-415-8480

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