



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

Errata

NUREG/CR-7177, "Compendium of Analyses to Investigate Select Level 1 Probabilistic Risk Assessment End-State Definitions and Success Criteria Modeling Issues," published May 2014

Note that some of the values in Table 2.4.1a/b are safety analysis assumption values, rather than Technical Specification values, physical tank sizes, etc. For instance, the cited values for "RWST total deliverable water volume" are smaller than the actual as-operated minimum Refueling Water Storage Tank water volumes for these plants.

Also note that the assumptions of reactor coolant pump trip behavior during loss of direct current bus 111 scenarios for Byron Unit 1 (i.e., that all four pumps would be tripped simultaneously from the main control room) are simpler than the actual situation at the as-designed, as-operated plant. More realistic trip assumptions can be seen in Table 22 of NUREG-2187, "Confirmatory Thermal-Hydraulic Analysis to Support Specific Success Criteria in the Standardized Plant Analysis Risk Models – Byron Unit 1."