

From: [Guzman, Richard](#)
To: ["Wanda D Craft \(Generation - 6\)"](#)
Subject: Millstone 2 - Request for Additional Information - Relief Request RR-04-19 Examination Cat R-A, Risk-Informed Piping Examinations (MF6569)
Date: Tuesday, March 08, 2016 4:09:30 PM

Wanda,

The NRC staff has reviewed the information provided in the subject request for relief dated July 30, 2015 (ADAMS Accession No. ML15216A359), and has determined that additional information is needed to complete its review. Shown below are the NRC staff's request for additional information questions. Please provide your formal response by April 22, 2016. If you have any questions, please contact me.

Thanks,

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**Rich Guzman**  
**Sr. Project Manager**  
**NRR/DORL**  
**US NRC**  
**301-415-1030**

**REQUEST FOR ADDITIONAL INFORMATION**  
**BY THE OFFICE OF NUCLEAR REACTOR REGULATION**  
**REQUEST FOR RELIEF RR-04-19**  
**DOMINION NUCLEAR CONNECTICUT INC**  
**MILLSTONE POWER STATION, UNIT 2**  
**DOCKET NO. 50-336**  
**TAC NO. MF6569**

By submittal dated July 30, 2015, (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15216A359) Dominion Nuclear Connecticut, Inc. (DNC, the licensee), submitted Requests for Relief RR-04-19 from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (B&PV Code), Section XI which requires welds in the Risk-Informed Inservice Inspection program to be volumetrically examined for the fourth 10-year inservice inspection interval.

The U.S. Nuclear Regulatory Commission (NRC) staff requests that the licensee provide additional information to support the relief from ASME Code Section XI-required volumetric examination of essentially 100% of the full-penetration welds.

For All Welds:

1. The piping and components are described as being constructed from "Stainless" or

“Carbon” steel, but the alloys are not identified. For each weld, please provide the materials of construction for the pipes, weld metals, and welded components.

2. The NRC staff notes that due to recent operating experience regarding thermal fatigue cracking in some plants, the Electric Power Research Institute (EPRI) issued an interim guidance, “EPRI-MRP Interim Guidance for Management of Thermal Fatigue (Accession Number ML15189A100),” that supplemented the existing industry thermal fatigue guidelines (e.g., MRP-146 and MRP-192) to better manage thermal fatigue cracking. Discuss whether DNC will take any compensatory measures to better manage thermal fatigue cracking in the subject welds to ensure structural integrity and leak tightness since essentially 100% coverage was not achieved by the ultrasonic test.