



Prairie Island Nuclear Generating Plant
1717 Wakonade Drive East
Welch, MN 55089

MAR 02 2016

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10 CFR 50.55a

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Unit 2
Docket 50-306
Renewed License No. DPR-60

Inservice Inspection Summary Report Unit 2. Refueling Outage Dates: 10/17/2015 to 12/06/2015, Fuel Cycle 28: 01/05/2014 to 12/06/2015

During the 2015 Prairie Island Nuclear Generating Plant (PINGP) Unit 2 refueling outage (2R29), inservice inspection examinations were performed for the third period of the fourth interval. Other examinations were performed for the first period of the fifth interval. The fourth interval was extended through the end of 2R29 as allowed by IWA-2430(d) of the 1998 Edition with 1999 and 2000 Addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Division 1.

The Owner's Activity Report (Form OAR-1) for the fourth interval examinations (Enclosure 1) is submitted in accordance with ASME Code Case N-532-4. Form OAR-1 for the fifth interval examinations (Enclosure 2) is submitted in accordance with ASME Code Case N-532-5. Each Form OAR-1 includes a listing of items with flaws or relevant conditions that exceeded the acceptance criteria of Division 1 and that required evaluation to determine acceptability for continued service (Table 1) and an abstract of repair/replacement activities that were required due to an item containing a flaw or relevant condition that exceeded Division 1 acceptance criteria (Table 2).

Summary of Commitments

This letter contains no new commitment and no revision to an existing commitment.

Kevin Davison
Site Vice President
Prairie Island Nuclear Generating Plant
Northern States Power Company - Minnesota

Enclosures

1. Form OAR-1, Fourth Interval, Report Number 2R29A
2. Form OAR-1, Fifth Interval, Report Number 2R29B

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cc: Administrator, Region III, USNRC
Project Manager, Prairie Island, USNRC
Resident Inspector, Prairie Island, USNRC
State of Minnesota

ENCLOSURE 1

**Owner's Activity Report (Form OAR-1),
Fourth Interval, Report Number 2R29A**

(2 pages follow)

FORM OAR-1 OWNER'S ACTIVITY REPORT

Prairie Island Nuclear Generating Plant

As required by the provisions of ASME Code Case N-532-4

Report Number 2R29A

Plant Prairie Island Nuclear Generating Plant, 1717 Wakonade Drive East, Welch, Minnesota, 55089

Unit No. 2 Commercial service date 12/21/1974 Refueling outage no. 2R29

Current inspection interval 4th ISI Inspection Interval for class 1, 2, and 3. 2nd IWE Inspection Interval for class MC

Current inspection period 3rd ISI Inspection Period for class 1, 2, and 3. 2nd IWE Inspection period for class MC

Edition and Addenda of Section XI applicable to the inspection plans 1998 Edition - 2000 Addenda for class 1, 2, and 3. 2001 Edition - 2003 Addenda for class MC

Date and revision of inspection plans 9/12/14 Rev. 20 for class 1, 2, and 3. 10/26/12 Rev. 9 for class MC

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans Same as Inspection Plans

Code Cases used: N-521, N-532-4, N-613-1, N-623, N-648-1

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this Owner's Activity Report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of refueling outage number 2R29 conform to the requirements of Section XI.

Signed Thomas Downing ISI Engineer Date 2/15/2016
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Minnesota and employed by The Hartford Steam Boiler Inspection and Insurance Company of Connecticut of Hartford Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

James R. DeF Commissions NB 12085 A,N,I MN 021924
Inspector's Signature National Board, State, Province and Endorsements

Date February 12, 2016

TABLE 1
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED
EVALUATION FOR CONTINUED SERVICE

Examination Category and Item Number	Item Description	Evaluation Description
B-A, B1.11 B-D, B3.90	Reactor Vessel Shell to Shell Reactor Vessel Nozzle to Shell	UT indications determined acceptable size under IWB-3500. AR 1512045.
B-N-1, B-N-3, B13.10, B13.70	Reactor Vessel Interior and Internals	Dents and scoring on non-pressure retaining parts. Cladding anomaly in SI nozzle. Indications determined to be non-relevant AR 1509158.
C-A, C1.30	22 Steam Generator	Preservice Ultrasonic Test (UT) Indication of a Flaw in 22 Steam Generator Tubesheet-to-Shell Weld. Accepted by evaluation and relief request. AR 1421771.
F-A, F1.20b	Aux Feed Pipe Hanger	Indication of hanger not matching drawing determined to be a non-relevant condition. AR 1498776.
F-A, F1.20a	Aux Feed Pipe Hanger	Indication of hanger not matching drawing determined to be a non-relevant condition. AR 1498780.
F-A, F1.20b	Aux Feed Pipe Hanger	Indication of a 0.3" gap between the baseplate and the wall at one corner of the baseplate. Non-relevant because outside the scope of Section XI. Determined acceptable under the Corrective Action Process. AR 1498787.
F-A, F1.20b	Aux Feed Pipe Hanger	Indication of hanger not matching drawing determined to be a non-relevant condition. AR 1500045.

TABLE 2
ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

Code Class	Plant ID #	Item Description	Description of Work Completed	Date Completed	R/R Plan #
3	075-012 4-ZH-1	122 Control Room Chiller Condenser	Replaced Tubes in the condenser, cut out and reinstalled piping line segment to facilitate tube replacement, and replaced piping flange fasteners.	4/3/2014	2-28-001
3	Spare	Spare Cooling Water Pump	Refurbished by weld repair and/or replacement of various pump parts.	6/3/2014	2-28-003

ENCLOSURE 2

**Owner's Activity Report (Form OAR-1),
Fifth Interval, Report Number 2R29B**

(3 pages follow)

FORM OAR-1 OWNER'S ACTIVITY REPORT

Prairie Island Nuclear Generating Plant

As allowed by the provisions of ASME Code Case N-532-5

Report Number 2R29B

Plant Prairie Island Nuclear Generating Plant, 1717 Wakonade Drive East, Welch, Minnesota, 55089

Unit No. 2 Commercial service date 12/21/1974 Refueling outage no. 2R29

Current inspection interval 5th ISI Inspection Interval for class 1, 2, and 3. 2nd IWE Inspection Interval for class MC

Current inspection period 1st ISI Inspection Period for class 1, 2, and 3. 2nd IWE Inspection period for class MC

Edition and Addenda of Section XI applicable to the inspection plans 2007 Edition - 2008 Addenda for class 1, 2, and 3. 2001 Edition - 2003 Addenda for class MC

Date and revision of inspection plans 12/11/14 Rev. 0 for class 1, 2, and 3. 10/26/12 Rev. 9 for class MC

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans Same as Inspection Plans

Code Cases used: N-532-5, N-716-1

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this Owner's Activity Report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of refueling outage number 2R29 conform to the requirements of Section XI.

Signed *Thomas Downing* ISI Engineer Date 2/15/2016
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Minnesota and employed by The Hartford Steam Boiler Inspection and Insurance Company of Connecticut of Hartford Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Donald R. Deady Commissions NB 12085 A,N,I MN 021924
Inspector's Signature National Board, State, Province and Endorsements

Date February 22, 2016

TABLE 1
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED
EVALUATION FOR CONTINUED SERVICE

Examination Category and Item Number	Item Description	Evaluation Description
B-G-2, B7.30	21 Steam Generator Manway Nuts	Light scratches on the face of nut 31 and a surface gouge on the face of nut 36 determined to be non-relevant. AR 1501102.
B-G-2, B7.50	Pressurizer Relief Valve	Indication of damaged threads determined non-relevant because outside the zone of thread engagement and less than one thread. AR 1498414.
B-P, B15.10 C-H, C7.10	Pressure Tests SP 2392 Bolted Connections	Boric acid indications at bolted connections in a system borated for the purpose of controlling reactivity accepted by evaluation. AR 1498694.
C-H, C7.10	Pressure Test SP 2168.13 Safety Injection System	Boric acid indication at bolted connection in a system borated for the purpose of controlling reactivity accepted by evaluation. AR 1502129.
D-B, D2.10	Control Valve	Wetness identified during pressure test determined to be packing leak. AR 1502772
F-A, F1.10c	RC Pipe Hanger	Loose Jam nut on 2-RRCH-283 discovered during repair/replacement activity. AR 1502572. Corrected under WO 535281.
F-A, F1.20a	CC Pipe Hanger	Broken CC pipe support discovered during RCS pressure test. Determined acceptable under the Corrective Action Process. AR 1503823.

TABLE 2
 ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

Code Class	Plant ID #	Item Description	Description of Work Completed	Date Completed	R/R Plan #
3	045-091	121 Motor Driven CL Pump	Replaced section of vent line piping.	6/2/2015	2-28-020
MC	2PENC-PAL	Personnel Air Lock	Weld repair the hinge restrictor bolt to the inner door surface of the Personnel Air Lock.	11/8/2015	2-28-021
2	274-013	23 CFCU Coils	Replaced the containment fan coil unit coils.	11/9/2015	2-28-036
2	274-011	21 CFCU Coils	Replaced the containment fan coil unit coils.	11/11/2015	2-28-037
2	274-014	24 CFCU Coils	Replaced the containment fan coil unit coils.	11/2/2015	2-28-039
2	274-012	22 CFCU Coils	Replaced the containment fan coil unit coils.	11/2/2015	2-28-040
3	10-2ZX-12	ZX System Piping Segment	Removed material affected by MIC and replaced with a flush patch using like for like material.	10/23/2015	2-28-041
1	2RC-10-1	21 PRZR RELIEF	Replaced inlet flange fasteners.	11/20/2015	2-28-052
1	234-011	21 Steam Generator	Replaced Primary Manway stud/nut assembly.	11/10/2015	2-28-057
1	234-011	21 Steam Generator	Installed several tube plugs due to vibrational wear noted during ET.	11/11/2015	2-28-059
1	234-012	22 Steam Generator	Plugged several tubes due to vibrational wear noted during ET.	11/11/2015	2-28-060
Yes	PI-182	Snubber	Replaced two lock nuts with standard nuts.	11/24/2015	2-28-067
Yes	PI-458	Snubber	Replaced one lock nut with a standard A563 nut. (Ref. EC-26476)	11/24/2015	2-28-068