

20 Inch Header

 $Combined \ Z \ and \ Y \ Direction \ Earthquake$

Moments at Joints (Global Coordinates)

Joint	X Direction Kip-Feet	Y Direction Kip-Feet	Z Direction Kip-Feet
1	0.44	0.08	0.17
2	0.21	0.20	0.05
3	0.20	0.18	0.28
4	0.11	0.06	0.04
5	0.31	1.06	0.51
6	0.12	0.35	0.05
7	0.00	0.69	0.23
8	0.00	0.03	0.12
9	0.00	1.69	0.65
10	0.00	0.03	0.12
11	0.00	0.69	0.23
12	0.12	0.35	0.05
13	0.31	1.06	0.51
14	0.11	0.06	0.04
15	0.20	0.18	0.28
16	0.21	0.20	0.05
17	0.44	0.08	0.17

MONTICELLO NUCLEAR GENERATION PLANT

20 Inch Header

Combined Z and Y Direction Earthquake

Forces at Joints (Member Coordinates)

Joint	Axial	Shear Y	Shear Z
	Kips	Kips	Kips
1	1.11	0.08	0.07
2	1.11	0.08	0.07
3	0.99	0.05	0.05
4	0.81	0.12	0.28
4 5	0.68	0.12	0.35
6	0.68	0.12	0.35
7	0.56	0.05	0.08
8	0.50	0.13	0.41
9	0.50	0.13	0.41
10	0.50	0.13	0.41
11	0.56	0.05	0.08
12	0.68	0.12	0.35
13	0.68	0.12	0.35
14	0.81	0.12	0.28
15	0.99	0.05	0.05
16	1.11	0.08	0.07
17	1.11	0.08	0.07



20 Inch Header

Combined Z and Y Direction Earthquake

Moment at Joints (Member Coordinates)

		*:	
Joint	Torsional	Bending Y	Bending Z
	Kip-Feet	Kip-Feet	Kip-Feet
1	0.19	0.08	0.44
2	0.19	0.20	0.11
3	0.19	0.18	0.27
4	0.09	0.06	0.08
5	0.09	1.10	0.60
6	0.09	0.36	0.10
7	0.09	0.69	0.23
8	0.00	0.03	0.12
9	0.00	1.68	0.65
10	0.00	0.03	0.12
11	0.09	0.69	0.23
12	0.09	0.36	0.10
13	0.09	1.10	0.60
14	0.09	0.06	0.08
15	0.19	0.18	0.27
16	0.19	0.20	0.11
17	0.19	0.08	0.44

MONTICELLO NUCLEAR GENERATION PLANT

20 Inch Header

Combined Z and Y Direction Earthquake

Pipe Support Reactions (Global Coordinates)

Forces at Points 5 and 13

X Direction 0.24 kips Y Direction 0.25 kips Z Direction 0.58 kips

Forces at Point 9

X Direction 0.00 kips Y Direction 0.26 kips Z Direction 0.83 kips



20 Inch Header

Combined Z and Y Direction Earthquake

Stresses at Joints

Joint	Stress
	Kips/Sq.In
1	0.053
2	0.033
3	0.037
4	0.016
5	0.135
6	0.041
7	0.078
8	0.013
9	0.196
10	0.013
11	0.078
12	0.041
13	0.135
14	0.016
15	0.037
16	0.033
17	0.053

REFERENCES

1. Chicago Bridge and Iron Company

20 Inch Header for Suppression Chamber Drawing 215 Rev. 5

Support Ass'y for 20 " Header Drawing 216 Rev.0

2. Design of Piping Systems, The M.W. Kellogg Company, Revised Second Edition, John Wiley & Sons, Inc.



IN REPLY REFER TO: 552 MISSION STREET SAN FRANCISCO, 94105 (415) 781.8914

December 24, 1968

General Electric Company Atomic Power Equipment Department 175 Curtner Avenue San Jose, California 95125

ATTENTION:

Mr. Ralph B. Gile

SUBJECT:

Monticello Nuclear Power Station

Pressure Suppression Chamber Dynamic Earthquake Analysis

Gentlemen:

Transmitted herewith is our report on the subject analysis. All pertinent information, calculations, and references are included.

Very truly yours,

H. J. Sexton and Associates, Engineers

Associate

REPORT ON THE

DYNAMIC EARTHQUAKE ANALYSIS

OF THE

PRESSURE SUPPRESSION CHAMBER

OF THE

MONTICELLO NUCLEAR POWER STATION

This report, prepared for the General Electric Company, presents the results of the seismic analysis of the Pressure Suppression Chamber for the Monticello Nuclear Power Station.

DESCRIPTION OF SUPPRESSION CHAMBER

The suppression chamber (Torus) is a torus-shaped steel vessel having an inside diameter of 27 feet 8 inches and a major diameter of 98 feet. It is supported vertically by 32 columns, 16 inner and 16 outer. Lateral stability is provided by four pinned, embedded anchorage assemblies, identified as seismic supports, which transmit seismic loads from the soffit of the torus to the concrete foundation. Dynamically the torus is a complete system in itself; the vents, headers, and downcommers are separated from the torus by means of bellows which provide no support.

METHOD OF ANALYSIS

The torus is idealized as a single degree of freedom system. Its spring constant is determined from the calculated shear deformations of the pins and bottom plates of the four seismic supports. By comparison the upper plates are rigid in shear, and all plates and pins are considered rigid in bending. The columns contribute a negligible amount of resistance compared to the stiffness of the seismic supports. The analysis is presented for the operating and flooded conditions.

Using the calculated stiffness and mass, the fundamental period of vibration of the torus is determined for the two cases considered. The seismic coefficient is read from the response spectrum for 1.0% damping.

2

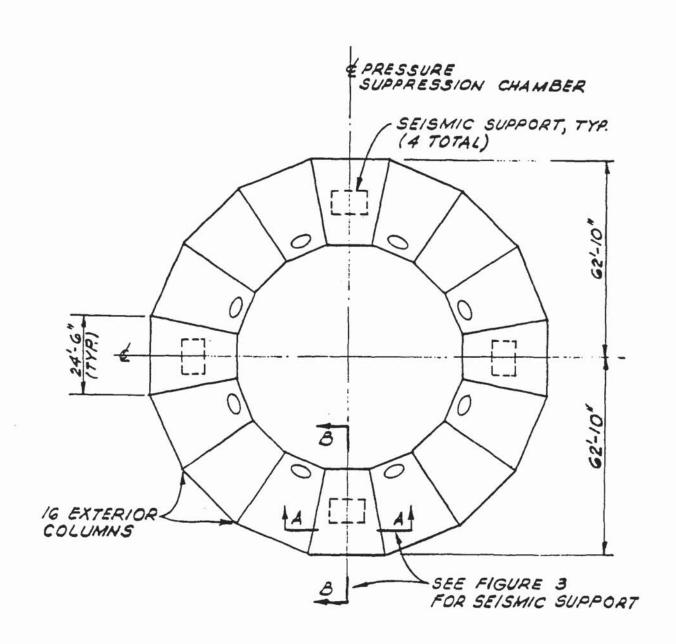
3

MONTICELLO NUCLEAR POWER STATION

PRESSURE SUPPRESSION CHAMBER

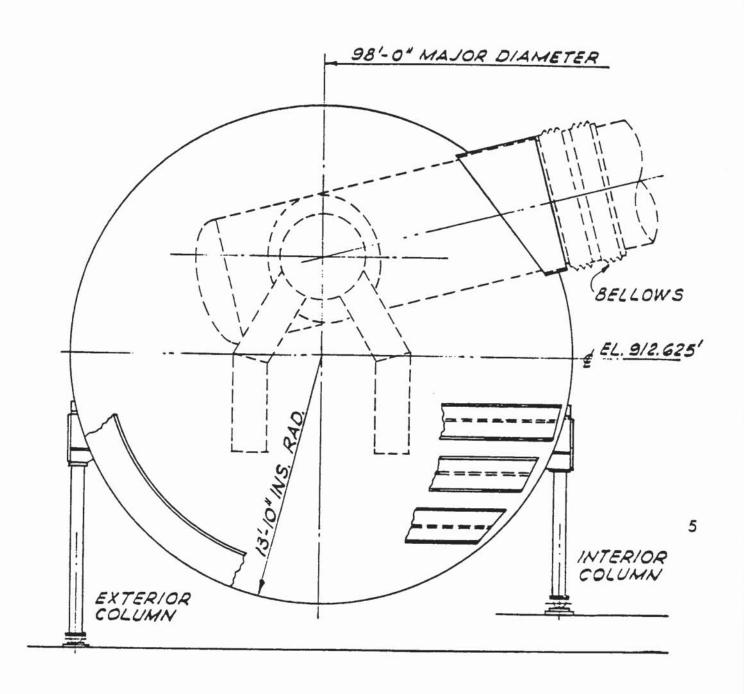
LIST OF FIGURES

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Plan of Suppression Chamber	ł
FIGURE 2:	
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FIGURE 3:	
Datails of Sajemic Supports	



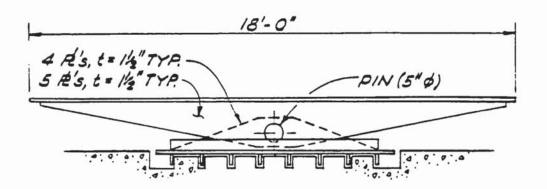
PLAN PRESSURE SUPPRESSION CHAMBER

FIGURE 1

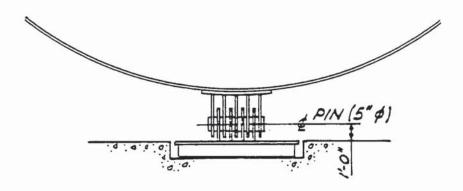


TYPICAL SECTION PRESSURE SUPPRESSION CHAMBER

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SECTION A-A



SECTION B-B

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JOB NO. 1229

JOB MONTICELLO NUCLEAR POWER STATION

BY GTQ DATE 11-15-68

CLIENT G.E.

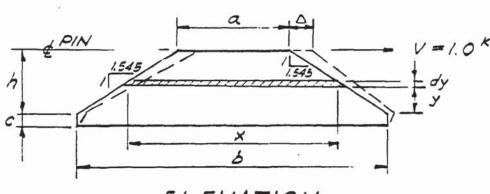
SUBJECT PRESSURE SUPPRESSION CHAMBER

CHK'D dB DATE 11-18-68

1. STIFFNESS OF CHAMBER SUPPORT

THE STIFFNESS OF EACH CHAMBER SUPPORT IS
PROVIDED BY A SET OF FIVE STIFFENER PLATES
AND PIN. ONLY SHEAR DEFORMATIONS
OF PLATE AND PIN ARE CONSIDERED SINCE
BENDING DEFORMATIONS ARE NEGLIGIBLE.
ONLY THE LOWER PLATES ARE CONSIDERED
SINCE THE UPPER PLATES ARE STIFF
COMPARED TO THE LOWER PLATES.

a. STIFFNESS OF LOWER PLATES



ELEVATION

$$\begin{array}{l}
t = PLATE \ THICKNESS \\
x = b - 2(1.545)y = b - 3.09y \\
A(y) = tx = t(b - 3.09y)$$

$$\Delta = k \frac{Vc}{GA} + k \int_{0}^{h} \frac{Vdy}{GA(y)} = \frac{kV}{G} \left[\frac{c}{A} + \int_{0}^{h} \frac{dy}{A(y)} \right]$$

$$\int_{0}^{h} \frac{dy}{A(y)} = \int_{0}^{h} \frac{dy}{t(b - 3.09y)} = \frac{1}{3.09t} \ln \left(\frac{b}{b - 3.09h} \right)$$

$$\Delta = \frac{kV}{Gt} \left[\frac{c}{b} + \frac{1}{3.09} \ln \left(\frac{b}{b - 3.09h} \right) \right]$$

A.7-7

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JOB MONTICELLO NUCLEAR POWER STATION BY GTQ DATE 11-15-68 JOB NO. /229 CLIENT G.E. SUBJECT PRESSURE SUPPRESSION CHAMBER CHK'D db DATE 1/-/8-68

STIFFNESS OF CHAMBER SUPPORT (CONT'D.)

INNER PLATES

FOR
$$G = 12 \times 10^{3} \text{ K/P/IN}^{2}$$
; $t = 2(1.5) = 3 \text{ IN}$.
 $K = G/5$; $b = G7 \text{ IN}$; $h = 9.875 \text{ IN}$; $C = 2.125 \text{ IN}$.

$$\Delta = \frac{1.2(1)}{12 \times 10^{3} \times 3} \left[\frac{2.125}{67.0} + \frac{1}{3.09} 2 n \frac{67}{67 - 30.5} \right] = .765 \times 10^{-5} \text{IN/K/P}$$

$$K = \frac{1}{\Delta} = \frac{1}{.765 \times 10^{-5}} = 1.31 \times 10^{3} \text{ K/P/IN}.$$

OUTER PLATES

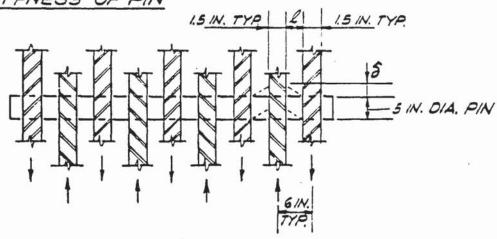
FOR
$$G = 12 \times 10^{3} \text{ KIP/IN}^{2}$$
; $t = 2(1.5) = 3 \text{ IN}$.
 $K = G/5$; $b = 72 \text{ IN}$; $h = 11.5 \text{ IN}$; $C = 0.5 \text{ IN}$.

$$\Delta = \frac{(1.2)(1)}{(12 \times 10^{3})(3)} \left[\frac{0.5}{72} + \frac{1}{3.09} \ln \frac{72}{72 - 35.5} \right] = 0.755 \times 10^{-5} \text{IN/KIP}$$

$$K = \frac{1}{\Delta} = 1.32 \times 10^{5} \text{ KIP/IN}.$$

8

& STIFFNESS OF PIN



PLAN

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JOB NO. 1229 JOB MONTICELLO NUCLEAR POWER STATION BY GTQ DATE 11-15-G8

SUBJECT PRESSURE SUPPRESSION CHAMBER CHK'D de DATE 11-18-48

STIFFNESS OF CHAMBER SUPPORT (CONT'D.)

$$S = \frac{KVL}{AG}$$
 $K_{PIN} = \frac{V}{S} = \frac{AG}{KL}$

$$K = 10/9 = 1.11$$

$$K_{PIN} = \frac{(1.96)(12 \times 10^3)}{(1.11)(1.5)} = 1.41 \times 10^5 \text{ KIP/IN.}$$

C. STIFFNESS OF EACH CHAMBER SUPPORT

$$\frac{1}{K_{S}} = \frac{1}{K_{P}} + \frac{1}{K_{P/N}} = \frac{1}{2.63 \times 10^{5}} + \frac{1}{1.41 \times 10^{5}} = 1.09 \times 10^{-5}$$

$$K_{S} = \frac{1}{1.09 \times 10^{-3}} = \frac{0.92 \times 10^{5} \text{ K/P}/\text{IN}}{1.41 \times 10^{5}} = 1.09 \times 10^{-5}$$

2. TORUS STIFFNESS KT

3. MASS

a. WATER WEIGHT

OPERATING CONDITION FLOODED CONDITION

5,223, KIPS 11.704.KIPS

b. STRUCTURE & EQUIPMENT

1,710. KIPS

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OB NO. 1229 JOB MONTICELLO NUCLEAR POWER STATION BY 8 DATE 11-18-68
CLIENT G. E. SUBJECT PRESSURE SUPPRESSION CHAMBER CHK'D DUH DATE 11-18-68



4. TORUS PERIOD & RESPONSE

ITEM	UNIT	OPERATING CONDITION	FLOODED
WATER WEIGHT	KIPS	5,223.0	11,704.0
STRUCTURE & EQUIP. WEIGHT	"	1,710.0	1,710.0
TOTAL WEIGHT, W	"	6,933.0	13,414.0
TORUS STIFFNESS, KT	K/IN.	1.84 × 105	1.84 × 105
PERIOD 7 = $2\pi \sqrt{\frac{W}{gK_T}}$	SEC.	0.062	0.087
$Sa^*\lambda = /\%$	9	0.07	0.15
SEISMIC FORCE F = W X Sa	KIPS	485.	2,010.0
TORUS DEFLECTION $\Delta_7 = \frac{F}{K_T}$	IN.	0.00264	0.0109

^{*}Sa FROM RESPONSE SPECTRUM CURVE.

5. MAX. SEISMIC FORCE AT EACH CHAMBER SUPPORT

$$F_{MAX} = K_S \triangle_T$$

10

REFERENCES

1. Chicago Bridge & Iron Company

General Plan, dated 9-5-67 VPF No. 1812-67-4 (General Electric Co.) EP No. 16-11 (General Electric Co.)

2. Chicago Bridge & Iron Company

Earthquake Ties, dated 4-7-67 VPF No. 1812-12-7 (General Electric Co.) EP No. 16-V (General Electric Co.)

3. Chicago Bridge & Iron Company

Suppression Chamber, General Plan and Field Assembly, dated 6-16-67 VPF No. 1812-68-5 (General Electric Co.) EP No. 16-11 (General Electric Co.)

MONTICELLO

RECIRCULATION LINES

METHODS OF ANALYSIS

(Code Requirement Section III, Appendix C-1320)

The following is a description of the computer programs used in the subject stress analysis and a brief description of their assumptions and theory. All programs conform to the design and control measures required by Appendix B of 10 CFR Part 50.

PROGRAM THEORY AND ASSUMPTIONS

ME-101

PURPOSE

The stresses and loads in piping systems due to restrained expansion, dead weight, seismic movement and earthquake are calculated using the ME-101 computer program.

METHOD OF ANALYSIS

ME-101 is a finite element computer program which performs linear elastic analysis of piping systems using standard beam theory techniques. ME-101 may be used for static and seismic load analysis of piping systems and also performs effective weight calculations.

Static analysis considers one or more of the following: thermal expansion, dead weight, uniformly distributed loads, and externally applied forces, moments, displacements and rotations, or individual force loads.

Seismic analysis is based on standard normal mode techniques and uses response spectrum data. Three methods of eigenvalue solution are available. Both Determinant Search and Subspace Iteration consider all data points as mass points. Kinematic Reduction considers masses only at specified data points in designated directions. Differential seismic anchor movement analysis and static seismic analysis are also provided.

REFERENCES

- K. Bathe, E. Wilson, F. Peterson, "SAP IV Structural Analysis Program for Static and Dynamic Response of Linear Systems," U. of California, Berkeley, Report No. EERC73-11, June 1973.
- 2. J. Gere, W. Weaver, "Analysis of Framed Structures," New York, D. Van Nostrand, 1965.
- 3. BSAP Theoretical Manual, Vol. 1.
- 4. R. Roark, "Formulas for Stress and Strain," New York, McGraw-Hill, 1965.
- 5. J. Gere, "Moment Distribution," Princeton, N.J., D. Van Nostrand, 1963.
- 6. K. Bathe, E. Wilson, "Numerical Methods in Finite Element Analysis," Englewood Cliffs, N.J., Prentice Hall, 1976.

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P.11n9.1 3/76

9.1189.2 3/78

- G. Stewart, "Error Bounds for Approximate Invariant Subspaces of Closed Linear Operators," SIAM
 J. of Numerical Analysis, Vol. 8, No. 4, December 1971.
- 8. G. Stewart, "Error and Perturbation Bounds for Subspaces Associated with Certain Eigenvalue Problems," SIAM J. of Numerical Analysis, Vol. 15, No. 4, October 1973.
- 9. G. Stewart, U. of Maryland Department of Computer Science, letter to R. Blum, November 22, 1977 (Attachment 1).

ME-101 STATIC ANALYSIS

For gravity, thermal and seismic movement analyses, the static load and displacement matrices are formed in addition to the stiffness matrix of the mathematical model. These matrices include the applied end forces and displacements, the distributed loading on the mathematical model, and the thermal forces developed in the members of the model, whichever is applicable. Once these matrices are formed, the system equilibrium equation is solved for U using the SESOL linear equation solver (see Ref. 6).

in which:

R = End force matrix

K = Stiffness matrix of piping

U = End displacement matrix

F = Fixed end force matrix

After the end displacements are determined, the individual member forces are obtained by using the member stiffness properties, and finally, the support reactions are calculated.

DYNAMIC ANALYSIS

The dynamic analysis of flexible piping systems is performed using the response spectrum method. A flexible piping system is idealized as a mathematical model consisting of lumped masses connected by massless elastic members. The lumped masses are carefully located so as to adequately represent the dynamic and classic properties of the piping system. The three-dimensional stiffness matrix of the mathematical model is determined by the direct stiffness method. Axial, shear, flexural and torsional deformations of each member are included. For curved members, a decreased stiffness is used in accordance with the Code. The mass matrix is also calculated.

After the stiffness and mass matrix of the mathematical model are calculated, the natural frequencies of piping system and corresponding mode shapes are determined using the following equation:

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where.

$$MU_{\xi} + KU_{\xi} = -a_{\xi}M$$
 I_{ξ} , $\xi = X, Y \text{ or } Z$

K = stiffness matrix

 $U_{\xi}\ddot{U}_{\xi}$ = displacement and accelerations due to the ground acceleration a_{ξ} , ξ = X, Y, or Z \overline{M} = mass matrix

 I_X = vector with I's in positions corresponding to δ_X displacements, O's elsewhere

ly , lz = same as lx except l's are in positions corresponding to δ_Y and δ_Z directions respectively

 a_X , a_Y , a_Z = ground accelerations in X, Y and Z directions

The equation of motion is solved via modal analysis, i.e.

$$U_{\xi} = \sum_{\text{modes}} \eta_{i} \Phi_{i}$$
;

where, $\Phi_i = i^{th}$ mode shapes

 η_i = generalized displacement of the ith mode shape

Substituting the modal formulation of U_{ξ} into the equation of motion and pre-multiplying by Φ_{j}^{T} and enforcing orthogonality, i.e.

$$\Phi_i^T M \Phi_i = \Phi_i^T K \Phi_i = 0 \quad i \neq j$$

yields the modal equation of motion

$$\ddot{\eta} + \omega_j^2 \eta_j = \Gamma_j a_{\xi}$$

where, $\omega_j^2 = \frac{\Phi_i^T \overline{M} \Phi_i}{\Phi_i^T \overline{M} \Phi_i} = \text{eigenvalue}$

=
$$(2\pi \cdot \text{frequency})^2$$

$$\Gamma_{j} = \frac{-\Phi_{j}^{\top} \overline{M} \left\{ I_{\xi} \right\}}{\Phi_{j}^{\top} \overline{M} \Phi_{j}} = \text{participation factor}$$



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The maximum generalized accelerations, η_{j} max; and displacements, η_{j} max = η_{j} max/ ω^{2} , are determined by a response spectra curve for the input ground motion.

The responses of the piping system with frequencies greater than ω cut-off are neglected.

The modal displacements and element end forces are:

displacement:
$$U_j = \eta_j \Phi_j$$

Two options are available for modal summation. They are square root of the sum of the squares (SRSS) and summation of closely spaced modes, via Regulatory Guide 1.92 Eqn. 4 (CS4).

VERIFICATION

The program has been verified by comparing its output with the ASME Benchmark Problem No. 1 as described in "Pressure Vessel and Piping 1972 Computer Programs Verification". The results were acceptable. Additional test problems are given in "Verification Report on ME-101, Linear Elastic Analysis of Piping Systems" Revision 1, February 1977, Bechtel Power Corporation.



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			PROJ	PROJECT NO	MONTICELLO I			SMEET 1 OF
CALC.	CALC. TYPE .	LOADING	<u>*</u>	DATE	TITLE, SERVICE OR SYSTEM DESCRIPTION	SUPER- CEDES CALC. NO.	SUPER. CEDED BY CALC. NO.	STATUS/REMARKS
58-10040 -551A	-	HYDRO 1EST	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. 1
SR-10040	-	MORMAL	ANDRENS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. 1
5R-10040 -SSIA	-	SCRAM	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. I
-551A	-	SHUTDOWN 1	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. YOL. 1
58-10040 -551A	-	SHUTDOWN 2	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. I
58-10040 -551A	-	SHUTDOWN 3	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. 1
10040	-	LOSS OF FW PUMP 1	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. YOL. I
-551A	-	LOSS OF FW PUMP 2 ANDREWS	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F, VOL. 1
SR-10040 -SSIA	-	LOSS OF FW PUMP 3 ANDREWS	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. YOL. I
SR-10040 - SSIA	-	RPV OVERPRES 1	AMDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. 1
SR-10040.	1	RPV OVERPRES 2	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. 1
58-100401 -551A	3	WEIGHT	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. VOL. 1
SR-10040 -SSIA	S	SEISHIC Y	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F. YOL. 1
10040	\$	SEISHIC X	ANDREWS	12-17-84	RECIRCULATION/RIIR - LOOP A			APPENDIX F. VOL. I
SR-10040	\$	SEISHIC Z	ANDREWS	12-17-84	RECIRCULATION/RIR - LOOP A			APPENDIX F. VOL. 1
SR-10040	s	\$61508	ANDREWS	12-17-84	RECIRCULATION/RHR - 100P A			APPENDIX F. VOL. 1
SR-10040	2	5£1508	ANDREWS	12-17-84	RECIRCULATION/RHR - LOOP A			APPENDIX F, VOL. 1
-10040 S18	1	HYDRO TEST	ANDREWS	12-19-84	RECIRCULATION/RIIR - LOOP B			APPENDIX F. VOL. 11
SR-100401	1	NORMAL	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, VOL. 11
58-10040 -5518	-	SCRAM	ANDREWS	1,2-19-84	RECIRCULATION/RIR - LOOP B			APPENDIX F, VOL. 11
- 10040	1	SHUT DOWN 1	ANDREWS	12-19-84	RECIRCULATION/RIR - 100P B			APPENDIX F. VOL. 11
5P-10040	-	SHUTPAM 2	ANDREWS	12-19-84	RECIRCULATION/RUR - 100P B			APPENDIX F. VOL. 11
SR-10040	,							

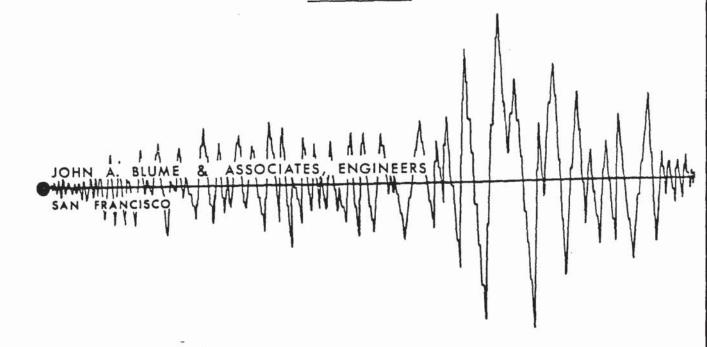
PIPING STRESS CALCULATION INDEX

PO. TYPE .						-	
1	CONDITION	ž.	DATE	TITLE, SERVICE ON SYSTEM DESCRIPTION	CEDES CALC.	CALC.	STATUS/MEMARKS
	LOSS OF FW PUMP I ANDREWS	ANDREWS	12-19-84	RECIRCULATION/RIR - LOOP B			APPENDIX F, VOL. 11
-5518 T	LOSS OF FIN PUMP 2 ANDREWS	ANDRENS	12-19-84	RECIRCULATION/RIIR - LOOP B			APPENDIX F, VOL. 11
-	LOSS OF FW PUMP 3 ANDREWS	ANDREWS	12-19-84	RECIRCULATION/RHR - 100P 8			APPENDIX F, VOL. 11
SR-10040 T	RPV OVERPRES 1	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP 8			APPENDIX F, VOL. 11
-	RPV OVERPRES 2	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, YOL. 11
>	WEIGHT	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, VOL. 11
~	SEISMIC Y	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, VOL. 11
~	SEISNIC X	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, VOL. 11
-55 l8 s	SEISHIC Z	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, VOL. 11
~	861308	ANDREWS	12-19-84	RECIRCULATIOM/RIIR - LOOP B			APPENDIX F, VOL. 11
~	SEISOB	ANDREWS	12-19-84	RECIRCULATION/RHR - LOOP B			APPENDIX F, VOL. 11
-551C TT		HARH	1-23-85	RECIRCULATION/RHR			APPENDIX F, VOL. 111-V
-5510 CLASS 1		HARI	1-23-85	RECIRCULATION/RHR - LOOP A			APPENDIX E, VOL. 1
SR-10040 SECT 111		HORI	1-23-85	RECIRCULATION/RIIR - LOOP 8			APPENDIX E, VOL. 11
	SHUTDOWN 4	ANDREWS	12-17-84	RECIRCULATION/PAIR - LOOP A			APPENDIX F. VOL. 1
-5518 T	SHUTDOWN 4	ANDREWS	12-19-84	RECIRCULATION/RUR - LOOP B			APPENDIX F. VOL. 11
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GENERAL ELECTRIC COMPANY Atomic Power Equipment Department

MONTICELLO NUCLEAR GENERATION PLANT-UNIT 1

Earthquake Analysis: Off-Gas Stack



JUHN A. BLUME & ASSOCIATES ENGINEERS

610 HOW ARD STREET + SAME FRANCISCO CALL . . .

December 10, 1968

General Electric Company Atomic Power Equipment Department 175 Curtner Avenue San Jose, California 95125

ATTENTION: Mr. R. B. Gile

MC - 750

MONTICELLO Nuclear Generation Plant - Unit 1 SUBJECT:

Earthquake Analysis:

Off-Gas Stack

Gentlemen:

Transmitted herewith is the subject report based on the information furnished by General Electric Company.

The results and recommendations presented herein are intended to be used in conjunction with the normal service loads in the final design calculations.

Very truly yours,

JOHN A. BLUME & ASSOCIATES, ENGINEERS

RSV:jl

MONTICELLO NUCLEAR GENERATION PLANT-UNIT 1 EARTHQUAKE ANALYSIS:

OFF-GAS STACK

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MONTICELLO NUCLEAR GENERATION PLANT-UNIT 1

EARTHQUAKE ANALYSIS:

OFF-GAS STACK

INTRODUCTION

This report presents the results of our investigations of the design earthquake response of the off-gas stack for the Monticello Nuclear Generation Plant-Unit 1. Based upon the recommended earthquake design criteria established for the plant, design envelopes of the maximum accelerations, displacements, shears, and overturning moments versus the height of the stack have been developed and are presented herein.

DESIGN CRITERIA

Based upon data developed by John A. Blume & Associates, Engineers, the design earthquake used in this analysis is the North 69° West component of the July, 1952 Taft earthquake, normalized to a maximum ground acceleration of 0.06 gravity. The earthquake design criteria for the Monticello Nuclear Generation Plant is contained in Reference 1.

DESCRIPTION OF STACK

The off-gas stack is a 320-foot-high reinforced concrete structure, having an internal diameter of 6.0 feet at the top and 32.0 feet at the base, with a 4.0-foot-thick octagonal foundation. The thickness of the concrete shell of the stack varies from 12 inches at the base to 7 inches at the top. The physical characteristics of the subject stack are described in Reference 2, and are schematically shown in Figure 1.

METHOD OF ANALYSIS

The off-gas stack was treated as a flexible cantilever and was idealized as a mathematical model consisting of nineteen lumped

masses connected by weightless elastic columns. The soil-structure interaction has been considered through the application of base translational and rotational springs. The complete mathematical model (Figure 2) shows the location and magnitude of the lumped masses, area and moment of inertia of the connecting columns, and the values of the base springs. The values for the base springs are based on the subsurface geotechnical properties of the material supporting the stack, and are listed in References 3, 4, and 5.

The elastic properties of the columns and the coupled action of the base springs were used to determine the flexibility matrix for the mathematical model. The flexibility calculations included the effects of flexural and shear deformations. Periods and mode shapes were determined using the flexibility matrix and the mass matrix.

The model was then subjected to the design acceleration timehistory at the base to obtain time-histories for accelerations, displacements, shear forces, and overturning moments at the various mass point elevations. These records were then scanned to determine the maximum values, which are graphically presented in Figures 3 through 6.

DESCRIPTION OF COMPUTER PROGRAM

The computer program used in this analysis was developed specifically to solve for dynamic response of structures subjected to arbitrary ground motions. Forms of input data used for the program include moments of inertia, effective shear area for the members, values of the base springs, weights of the lumped masses, and the input acceleration time-history.

The computer retains the response of each mass for each individual mode at each increment of time, and the total response for each increment of time is obtained by adding together the response of each mass point for each mode at a particular instant of time.

The result is an exact combination of mode participation which does not require approximate methods such as the root-mean-square method.

DISCUSSION OF RESULTS

The envelopes of maximum accelerations, shears, moments, and displacements are presented in Figures 3 through 6.

The calculations previously described were performed with the aid of a digital computer. The influence of $7\frac{th}{}$ and higher modes of vibration was considered negligible, and therefore ignored in the response calculations. A damping value of 5% was assigned to all modes.

The first six natural periods of vibration are listed below:

First Mode1.131	seconds
Second Mode	seconds
Third Mode0.171	seconds
Fourth Mode0.108	seconds
Fifth Mode	seconds
Sixth Mode 0.061	seconds

RECOMMENDATIONS

It is recommended that the off-gas stack be designed to resist the seismic shears and moments presented herein. The stack should be stable against an overturning moment of 14,312 kip-ft. In addition, the structure should be reviewed for safe shutdown requirements.

MONTICELLO NUCLEAR GENERATION PLANT-UNIT 1 EARTHQUAKE ANALYSIS: OFF-GAS STACK

REFERENCES

- "Monticello Nuclear Generation Plant, Recommended Earthquake Criteria," John A. Blume & Associates, Engineers, July 15, 1966.
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- "Monticello Nuclear Generation Plant, Earthquake Analysis
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- 4. "Foundation Investigation Proposed Nuclear Power Plant -Unit Number 1, Monticello, Minnesota," Dames & Moore, July 27, 1966 (includes Supplements 1 through 5).
- "Dynamic Response Data Investigation Proposed Nuclear Power Plant, Monticello, Minnesota," Dames & Moore, July 7, 1966.

Design Reconciliation

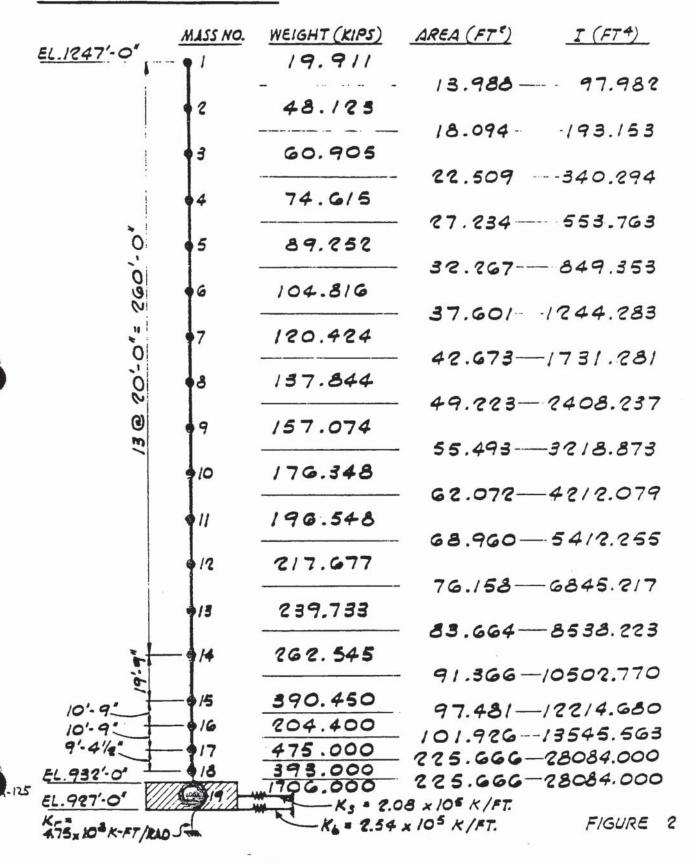
A design basis review of the Offgas Stack identified several differences between the designed configuration of the Offgas Stack and the analyzed configuration presented within this report. An engineering review of these differences concluded that the dynamic results presented herin are sufficiently accurate for design purposes.

JOHN A. OLDME AND ASSOCIATES, ENGINEERS MONTICELLO NUCLEAR GENERATION PLANT

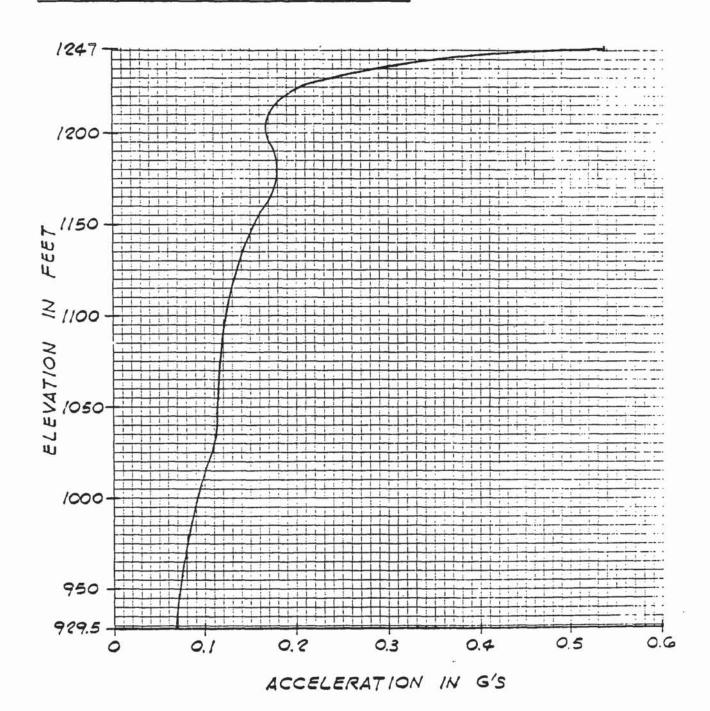
OFF-GAS STACK *OFL.1260'-0" m EL.1247'-0" PLAN @ EL. 932'-6" EL.968'-0" PLAN @ EL. 946'-6" SCALE: 1"= 20' EL 946'-6" SECTION SCALE: 1"=40' FIGURE 1

55'-0" (CCTAGON)

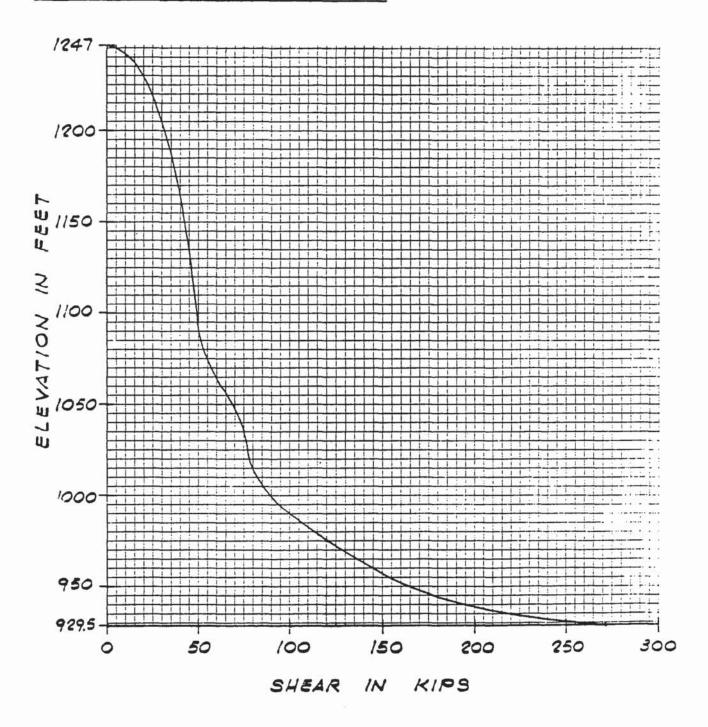
MONTICELLO NUCLEAR GENERATION PLANT OFF-GAS STACK



MONTICELLO NUCLEAR GENERATION PLANT OFF-GAS STACK ACCELERATION DIAGRAM UNDER SEISMIC LOADS

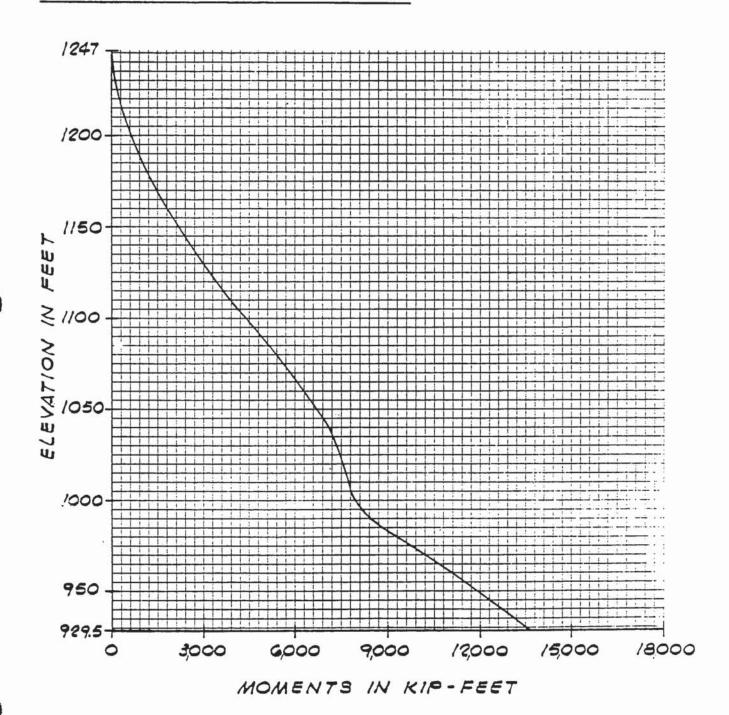


MONTICELLO NUCLEAR GENERATION PLANT OFF-GAS STACK DESIGN SHEAR DIAGRAM UNDER SEISMIC LOADS





MONTICELLO NUCLEAR GENERATION PLANT OFF-GAS STACK DESIGN MOMENT DIAGRAM UNDER SEISMIC LOADS



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MONTICELLO NUCLEAR GENERATION PLANT OFF-GAS STACK DISPLACEMENT DIAGRAM UNDER SEISMIC LOADS

