



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 1, 2016

Mr. Dennis L. Koehl
President and CEO/CNO
STP Nuclear Operating Company
South Texas Project
P.O. Box 289
Wadsworth, TX 77483

SUBJECT: SOUTH TEXAS PROJECT, UNITS 1 AND 2 – REQUEST FOR ADDITIONAL INFORMATION RELATED TO A LICENSE AMENDMENT REQUEST TO EXTEND THE CONTAINMENT INTEGRATED LEAK RATE TEST FREQUENCY (CAC NOS. MF6176 AND MF6177)

Dear Mr. Koehl:

By letter dated April 29, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15128A352), as supplemented by letters dated June 29, October 8, and November 11, 2015 (ADAMS Accession Nos. ML15198A147, ML15293A509, and ML15329A304, respectively), STP Nuclear Operating Company (the licensee) requested changes to the Technical Specifications for South Texas Project, Units 1 and 2. The proposed change would permit the existing Containment Integrated Leak Rate Testing frequency to be extended from 10 years to 15 years on a permanent basis.

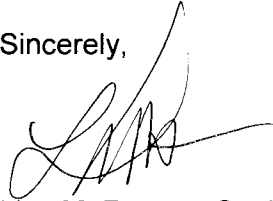
The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the information provided in your application and determined that additional information, as described in the enclosure to this letter, is required to complete the review.

A draft version of the request for additional information (RAI) was provided to Mr. Lance Sterling of your staff via e-mail on February 18, 2016 (ADAMS Accession No. ML16050A026). The questions were discussed with your staff during a conference call the same day. The RAI was revised and a draft copy of the enclosed request for additional information (RAI) was provided to Mr. Lance Sterling of your staff via e-mail on February 24, 2016 (ADAMS Accession No. ML16057A032). The questions were discussed with your staff during a conference call on February 25, 2016. Your response is requested by March 21, 2016, to allow the NRC staff enough time to make a regulatory decision on your license amendment request by your requested date of April 30, 2016. If you are unable to provide a satisfactory RAI response by March 21, 2016, the staff is unlikely to meet your requested completion date.

D. Koehl

If you have any questions, please contact me at 301-415-1906 or via e-mail at Lisa.Regner@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read 'L. Regner', with a long horizontal flourish extending to the right.

Lisa M. Regner, Senior Project Manager
Plant Licensing Branch IV-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-498 and 50-499

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION
LICENSE AMENDMENT REQUEST TO EXTEND THE
CONTAINMENT INTEGRATED LEAK RATE TEST FREQUENCY
STP NUCLEAR OPERATING COMPANY
SOUTH TEXAS PROJECT, UNITS 1 AND 2
DOCKET NOS. 50-498 AND 50-499

By letter dated April 29, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15128A352), as supplemented by letters dated June 29, October 8, and November 11, 2015 (ADAMS Accession Nos. ML15198A147, ML15293A509, and ML15329A304, respectively), STP Nuclear Operating Company (the licensee) requested changes to the Technical Specifications for South Texas Project (STP), Units 1 and 2. The proposed change would permit the existing Containment Integrated Leak Rate Testing frequency to be extended from 10 years to 15 years on a permanent basis.

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided in your application and determined that the following additional information is required to complete the review.

SCVB RAI:

The STP Updated Final Safety Analysis Report (UFSAR) Section 6.2.1.3, "Mass and Energy Release Analyses for Postulated Loss of Coolant Accidents," states the use of the Westinghouse WCAP-10325-P-A methodology for loss-of-coolant accident (LOCA) mass and energy release analysis. Westinghouse Electric Company LLC (Westinghouse) has issued Nuclear Safety Advisory Letters (NSALs)-06-6, -11-5, and -14-2, and InfoGram (IG)-14-1 to report errors in this methodology. Also, a new methodology (GOTHIC) is used for the LOCA containment analysis for which the mass and energy input needs to be corrected based on the above NSALs and the InfoGram.

The licensee indicated that the three Westinghouse NSALs were corrected and incorporated into the new LOCA containment analysis and the analysis results have been incorporated into the UFSAR.

1. Please describe any changes in inputs and assumptions in the reanalysis that could potentially reduce the conservatism in the previous analysis. Please describe which UFSAR sections and figures contain the revised results of the LOCA mass and energy releases, containment pressure response, and Equipment Qualification pressure/temperature response analyses. Provide a brief description of the changes from the previous analysis results.

Enclosure

2. The information provided in the IG-14-1 is generic. Please describe STP plant-specific analysis/evaluations performed to determine the applicability of the errors reported in the InfoGram to STP and its impact on the current analysis of record.

D. Koehl

If you have any questions, please contact me at 301-415-1906 or via e-mail at Lisa.Regner@nrc.gov.

Sincerely,

/RA/

Lisa M. Regner, Senior Project Manager
Plant Licensing Branch IV-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-498 and 50-499

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

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LPL4-1 Reading

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ADAMS Accession No. ML16053A187

*via email

OFFICE	NRR/DORL/LPL4-1/PM	NRR/DORL/LPL4-1/LA	NRR/DSS/SCVB/BC	NRR/DORL/LPL4-1/BC	NRR/DORL/LPL4-1/PM
NAME	LRegner (MWatford for)	JBurkhardt*	RDennig*	RPascarelli	LRegner
DATE	2/25/16	2/25/16	2/25/16	03/01/16	03/01/16

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