



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 23, 2016

Mr. Shane M. Marik
Site Vice President and Chief Nuclear Officer
Omaha Public Power District
Fort Calhoun Station
9610 Power Lane, Mail Stop FC-2-4
Blair, NE 68008

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 – REQUEST FOR ADDITIONAL
INFORMATION RE: REVISE CURRENT LICENSING BASIS TO ALLOW USE
OF EQUIPMENT CLASSIFICATION METHODOLOGY FROM
ANSI/ANS-58.14-2011 (CAC NO. MF6721)

Dear Mr. Marik:

By letter dated September 10, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15258A680), Omaha Public Power District submitted a license amendment request to revise the current licensing basis to allow the use of an equipment classification methodology from the American Nuclear Standards Institute/American Nuclear Society (ANSI/ANS)-58.14-2011 at Fort Calhoun Station, Unit No. 1.

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided in your application and determined that additional information is required in order to complete its formal review of your request. The enclosed questions were provided to E. Matzke of your staff on February 17, 2016. Please provide a response to the enclosed questions within 45 days of the date of this letter. If you have any questions, please contact me at 301-415-2296 or via e-mail at Fred.Lyon@nrc.gov.

Sincerely,

A handwritten signature in black ink that reads "CF Lyon".

Carl F. Lyon, Project Manager
Plant Licensing Branch IV-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

LICENSE AMENDMENT REQUEST

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

By letter dated September 10, 2015 (Agencywide Document Access and Management System (ADAMS) Accession No. ML15258A680), Omaha Public Power District (the licensee) submitted a license amendment request (LAR) to revise the current licensing basis to allow the use of an equipment classification methodology from the American Nuclear Standards Institute/American Nuclear Society (ANSI/ANS)-58.14-2011 at Fort Calhoun Station, Unit No. 1 (FCS).

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided in the application and determined that additional information is required in order to complete its formal review of the request.

RAI-1

Revised Section 4.0, "Seismic Classifications," of the proposed LAR states that there are three seismic classifications for FCS structures, systems, and components (SSCs): Seismic Category I, Seismic Category II, and Non-seismic. However, Section 2, "Seismic Criteria, Analysis and Instrumentation," of Appendix F of the Updated Safety Analysis Report (USAR) only addresses seismic criteria for Class I and Class II. The change of seismic classification from two classes to three classes should be consistent throughout the USAR. Please:

- (a) Indicate likely USAR revisions to capture the above comment;
- (b) Provide the seismic design criteria for Seismic Category II SSCs in the LAR and indicate likely USAR revisions to address this comment;
- (c) Identify the design control document that lists Seismic Category II SSCs.

RAI-2

The proposed LAR eliminates many sections, including safety class interfaces, correlations between safety class and equipment design code, and quality assurance (including Table N-2). Since the sections removed from the USAR will no longer be part of the current licensing basis for FCS, please provide justification for removing these sections from the licensing basis.

RAI-3

Table N-1, "Basic Design Requirements," of the proposed LAR states that pressure integrity Class C-4 (equivalent to safety class A) is either Seismic Category II or "no requirements specified." However, Section 2.0 of the proposed LAR states that safety class A is

Enclosure

non-safety-related with augmented requirements (i.e., those SSCs are not safety-related but are relied upon during a special event, such as station blackout, or to which a licensing requirement or commitment applies. Therefore, based on Table N-1, those non-safety related SSCs with augmented requirements may correspond to either Seismic Category II or “no requirements specified.” Please justify why safety class A or pressure integrity Class C-4 is designated as either Seismic Category II or “no requirements required.”

February 23, 2016

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Sincerely,

/RA/

Carl F. Lyon, Project Manager
Plant Licensing Branch IV-1
Division of Operating Reactor Licensing
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