

Entergy Nuclear Northeast Entergy Nuclear Operations, Inc. James A. FitzPatrick NPP P.O. Box 110 Lycoming, NY 13093 Tel 315-349-6024 Fax 315-349-6480

Brian R. Sullivan Site Vice President – JAF

JAFP-16-0002 February 4, 2016

United States Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555-0001

Subject: LER: 2015-006-01, Transitory Secondary Containment Differential Pressure Excursions James A. FitzPatrick Nuclear Power Plant Docket No. 50-333

Dear Sir or Madam:

This report is submitted in accordance with 10 CFR 50.73(a)(2)(v)(C).

License No. DPR-59

There are no commitments contained in this report.

Questions concerning this report may be addressed to Mr. Chris M. Adner, Regulatory Assurance Manager, at (315) 349-6766.

Sincerely,

Brian R. Sullivan Site Vice President

BRS/CMA/ds

- Enclosure(s): JAF LER: 2015-006-01, Transitory Secondary Containment Differential Pressure Excursions
- cc: USNRC, Region 1 USNRC, Project Directorate USNRC, Resident Inspector

102-2014) Estinded butte presports to carry with the interactive voltetion rouget. B Dama Science and Sc	NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSIO				N APP	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 01/31/2017						1/31/2017						
1. FACILITY NAME James A. FitzPatrick Nuclear Power Plant 2. DOCKET NUMBER 05000333 1 OF 5   4. TTLE Transitory Secondary Containment Differential Pressure Excursions 1 OF 5   5. EVEN DATE 6. LER NUMBER Transitory Secondary Containment Differential Pressure Excursions 8. OTHER FACILITIES INVOLVED N/A NOUTH   09 22 2015 2015 - 006 - 01 2 4 2016 N/A N/A   09 22 2015 2015 - 006 - 01 2 4 2016 N/A N/A   9. OPERATING MODE 11. THIS REPORT IS SUBMITTED PURSUANT TO THR EQUIREMENTS OF 10 CFR 5: (Check at that apphy) 0000ET NUMBER N/A 0000ET NUMBER N/A 0000ET NUMBER N/A 0000ET NUMBER N/A   1 0.20203(a)(1) 20.2203(a)(3) 50.73(a)(2)(1)(A) 50.73(a)(2)(1)(A) 50.73(a)(2)(1)(A)   1 20.2203(a)(2)(1) 50.36(c)(1)(1)(A) 50.73(a)(2)(1)(B)	LICENSEE EVENT REPORT (LER)						Estim Repo Send Branc intern Regu 2050: contro the in	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collection Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or b internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information an Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, D 20503. If a means used to impose an information collection does not display a currently valid OM control number, the NRC may not conduct or sponsor, and a person is not required to respond to the information collection.						t: 80 hours. to industry. Collections 0001, or by ormation and shington, DC ty valid OMB o respond to,				
James A. FitzPatrick Nuclear Power Plant   05000333   1 OF 5     A. TITLE     Transitory Secondary Containment Differential Pressure Excursions     A. TITLE     B. OTHER FACILITIES INVOLVED     MONTH   DATE   6. LER NUMBER   7. REPORT DATE   8. OTHER FACILITIES INVOLVED     MONTH   DATE   6. UER NUMBER   7. REPORT DATE   8. OTHER FACILITIES INVOLVED     MONTH   DATE   REPORT DATE   REPORT DATE   NO     OCCURT NUMEE   NO   NO   NO     0   20.2016   20.2203(a)(3)(i)   50.73(a)(2)(ii)(A)     20.2203(a)(2)(i) <th colspa<="" td=""><td>1. FACILIT</td><td>Y NAME</td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td>2. D</td><td>OCK</td><td>KET NUMBER</td><td></td><td>3. PAGE</td><td></td><td></td><td></td></th>	<td>1. FACILIT</td> <td>Y NAME</td> <td></td> <td></td> <td></td> <td></td> <td></td> <td></td> <td></td> <td>2. D</td> <td>OCK</td> <td>KET NUMBER</td> <td></td> <td>3. PAGE</td> <td></td> <td></td> <td></td>	1. FACILIT	Y NAME								2. D	OCK	KET NUMBER		3. PAGE			
4. TITLE Transitory Secondary Containment Differential Pressure Excursions   5. EVENT DATE 6. LER NUMBER 7. REPORT DATE 8. OTHER FACILITIES INVOLVED   MONTH DAY YEAR YEAR Securement N/A   09 22 2015 2015 0.06 -01 2 4 2016 N/A N/A   9. OPERATING MODE 11. THIS REFORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: C/Accurry NAME N/A   9. OPERATING MODE 11. THIS REFORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: C/Accurry NAME N/A   1 20.2201(d) 20.2203(a)(3)(i) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A)   1 20.2203(a)(1) 20.2203(a)(3)(i) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A)   10. POWER LEVEL 20.2203(a)(2)(ii) 50.36(c)(1)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A)   10. POWER LEVEL 20.2203(a)(2)(ii) 50.37(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(A) 50.73(a)(2)(0)(B) 50.73(a)(2)(0)(B) 50.73(a)(2)(	James A	۸. FitzP	'atrick	: Nuclea	r Powei	r Plan	nt			05	000	)333		1 C	)F 5			
Transitory Secondary Containment Differential Pressure Excursions     5. EVENT DATE   6. LER NUMBER   7. REPORT DATE   8. OTHER FACILITIES INVOLVED     MONTH   DAY   YEAR   YEAR   SEQUENTIAL NUMBER   REV NO.   MONTH   DAY   YEAR   SCOLET NUMBER N/A   DOCKET NUMBER N/A     09   22   2015   2015 - 006 - 01   2   4   2016   PACULY NME N/A   DOCKET NUMBER N/A     9. OPERATING MODE   11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR S: (Check all that apply)   20.2201(b)   20.2203(a)(3)(i)   50.73(a)(2)(i)(c)   50.73(a)(2)(vii)     1   20.2201(d)   20.2203(a)(3)(i)   50.73(a)(2)(i)(A)   50.73(a)(2)(vii)   50.73(a)(2)(vii)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(i)(A)   50.73(a)(2)(v)(A)   50.73(a)(2)(x)(A)     10. 20.2203(a)(2)(ii)   50.36(c)(2)   50.73(a)(2)(v)(A)   50.73(a)(2)(x)(A)   50.73(a)(2)(x)(A)     10. 20.2203(a)(2)(ii)   50.36(c)(2)   50.73(a)(2)(v)(A)   50.73(a)(2)(x)(A)   50.73(a)(2)(x)(A)   50.73(a)(2)(x)(A)         10. POWER LEVEL       20.220	4. TITLE																	
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09   22   2015   2015   006   -   01   2   4   2016   N/A   N/A     9. OPERATING MODE   11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)   0.073(a)(2)(v)(C)   50.73(a)(2)(v)(I)     1   20.2201(b)   20.2203(a)(3)(I)   50.73(a)(2)(V)(C)   50.73(a)(2)(v)(I)     20.2203(a)(1)   20.2203(a)(3)(I)   50.73(a)(2)(V)(I)   50.73(a)(2)(V)(I)     20.2203(a)(2)(I)   50.36(c)(1)(I)(A)   50.73(a)(2)(V)(I)   50.73(a)(2)(V)(I)     10. POWER LEVEL   20.2203(a)(2)(III)   50.36(c)(1)(I)(I)   50.73(a)(2)(V)(I)   50.73(a)(2)(V)(I)     10. 20.2203(a)(2)(III)   50.36(c)(1)(III)   50.73(a)(2)(V)(A)   50.73(a)(2)(V)(A)   50.73(a)(2)(V)(A)     10. POWER LEVEL   20.2203(a)(2)(III)   50.36(c)(1)(III)   50.73(a)(2)(V)(A)   50.73(a)(2)(V)(A)   50.73(a)(2)(V)(A)     10. POWER LEVEL   20.2203(a)(2)(IIII)   50.36(c)(1)(III)   50.73(a)(2)(V)(A)   50.73(a)(2)(V)(A)   50.73(a)(2)(V)(A)     10. POWER LEVEL   20.2203(a)(2)(VII)   50.73(a)(2)(V)(B)   57.73(a)(2)(V)(C)   OTHER     10. 20.2203(a	MONTH (	DAY `	YEAR	YEAR	SEQUENTI/ NUMBER	R R	IEV NO.	MONTH	DAY	YE/	AR				N N		UMBER	
9. OPERATING MODE   11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)     1   20.2201(b)   20.2203(a)(3)(i)   50.73(a)(2)(i)(C)   50.73(a)(2)(vii)     1   20.2201(d)   20.2203(a)(3)(i)   50.73(a)(2)(i)(A)   50.73(a)(2)(vii)(A)     20.2203(a)(1)   20.2203(a)(3)(i)   50.73(a)(2)(ii)(A)   50.73(a)(2)(vii)(A)     20.2203(a)(2)(i)   50.36(c)(1)(i)(A)   50.73(a)(2)(ii)(B)   50.73(a)(2)(vii)(B)     20.2203(a)(2)(ii)   50.36(c)(1)(i)(A)   50.73(a)(2)(vi)(A)   50.73(a)(2)(x)(A)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(i)(A)   50.73(a)(2)(v)(A)   73.71(a)(4)     20.2203(a)(2)(iii)   50.36(c)(2)   50.73(a)(2)(v)(B)   73.71(a)(5)     20.2203(a)(2)(v)   50.73(a)(2)(i)(A)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(vi)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   Specify in Abstr	09	22 2	2015	2015	- 006	; -	01	2	4	20	16				N		UMBER	
1   20.2201(b)   20.203(a)(3)(i)   50.73(a)(2)(i)(C)   50.73(a)(2)(vii)     20.2201(d)   20.2203(a)(3)(ii)   50.73(a)(2)(ii)(A)   50.73(a)(2)(viii)(A)     20.2203(a)(1)   20.2203(a)(3)(ii)   50.73(a)(2)(ii)(B)   50.73(a)(2)(viii)(B)     20.2203(a)(2)(i)   20.2203(a)(2)(i)   50.73(a)(2)(viii)(B)   50.73(a)(2)(viii)(B)     20.2203(a)(2)(i)   50.36(c)(1)(i)(A)   50.73(a)(2)(vi)(A)   50.73(a)(2)(x)(A)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(vi)(A)   50.73(a)(2)(x)(A)     10. 20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(v)(A)   73.71(a)(4)     20.2203(a)(2)(vi)   50.46(a)(3)(ii)   50.73(a)(2)(v)(B)   73.71(a)(5)     100   20.2203(a)(2)(vi)   50.73(a)(2)(i)(A)   95.73(a)(2)(v)(D)   Specify in Abstract below or in NCC Form 366A     100   20.2203(a)(2)(vi)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(D)   Specify in Abstract below or in NCC Form 366A     100   20.2203(a)(2)(vi)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(D)   Specify in Abstract below or in NCC Form 366A     110   20.2203(a)(2)(vi)   50.73(a)(2)(i)(B)	9. OPER/	ATING MO	ODE	<u>11. TI</u>	HIS REPO	<u>)RT IS </u>	SUBM	<u>ITTED PI</u>	JRSUA	NT TO	THE		<u>IS OF 10 CF</u>	R§: (Check	all tha	it app	ly)	
1   20.2201(d)   20.2203(a)(3)(ii)   50.73(a)(2)(ii)(A)   50.73(a)(2)(ii)(A)     20.2203(a)(1)   20.2203(a)(2)(ii)   50.73(a)(2)(ii)(B)   50.73(a)(2)(ii)(B)     20.2203(a)(2)(i)   50.36(c)(1)(i)(A)   50.73(a)(2)(ii)(B)   50.73(a)(2)(ii)(B)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(ii)(A)   50.73(a)(2)(ix)(A)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(iv)(A)   50.73(a)(2)(ix)(A)     10. POWER LEVEL   20.2203(a)(2)(iii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(iv)(A)   50.73(a)(2)(ix)(A)     10. POWER LEVEL   20.2203(a)(2)(iii)   50.36(c)(2)   50.73(a)(2)(iv)(A)   73.71(a)(5)     10. 20.2203(a)(2)(iv)   50.46(a)(3)(ii)   50.73(a)(2)(v)(B)   73.71(a)(5)     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(D)   Specify in Abstract below or in NRC Form 366A     11. CENSEE CONTACT   12. LICENSEE CONTACT FOR THIS LER   12. LICENSEE CONTACT FOR THIS LER   13.5349-6766     13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   14.8014   FACTURER   REPORTABLE     CAUSE   SYSTEM   C				20.2	.201(b)			20.2203	(a)(3)(i	)		50.73(a)(2	2)(i)(C)	50.7	3(a)(2)	a)(2)(vii)		
Image: Section of the sectio		1		20.2201(d) 20.2203(a)(1)			20.2203(a)(3)(ii) 20.2203(a)(4)			i)		50.73(a)(2)(ii)(A)		50.7	50.73(a)(2)(viii)(A)			
20.2203(a)(2)(i)   50.36(c)(1)(i)(A)   50.73(a)(2)(iii)   50.73(a)(2)(ix)(A)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(iv)(A)   50.73(a)(2)(x)(A)     10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(iv)(A)   50.73(a)(2)(x)     10. POWER LEVEL   20.2203(a)(2)(iii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(v)(A)   73.71(a)(4)     10. POWER LEVEL   20.2203(a)(2)(iv)   50.46(a)(3)(ii)   50.73(a)(2)(v)(B)   73.71(a)(5)     10. POWER LEVEL   20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(D)   Specify in Abstract below or in NRC Form 366A     12. LICENSEE CONTACT   Intervent Manager   TELEPHONE NUMBER (Include Area Code) 315-349-6766   315-349-6766     13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   REPORTABLE TO EPIX   CAUSE   SYSTEM   COMPONENT   FACTURER   REPORTABLE TO EPIX   NANU- FACTURER   REPORTABLE TO EPIX   NANU- FACTURER   REPORTABLE TO EPIX   NANU- FAC		•									50.73(a)(2)(ii)(B)		50.73	3(a)(2)	)(viii)(	В)		
10. POWER LEVEL   20.2203(a)(2)(ii)   50.36(c)(1)(ii)(A)   50.73(a)(2)(v)(A)   50.73(a)(2)(x)     100   20.2203(a)(2)(iii)   50.36(c)(2)   50.73(a)(2)(v)(A)   73.71(a)(4)     100   20.2203(a)(2)(iv)   50.46(a)(3)(ii)   50.73(a)(2)(v)(B)   73.71(a)(5)     20.2203(a)(2)(v)   50.73(a)(2)(i)(A)   50.73(a)(2)(v)(B)   73.71(a)(5)     20.2203(a)(2)(v)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(vi)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(D)   Specify in Abstract below or in NRC Form 366A     12. LICENSEE CONTACT   12. LICENSEE CONTACT FOR THIS LER   12. LICENSEE CONTACT FOR THIS LER     LICENSEE CONTACT   TELEPHONE NUMBER (Include Area Code)   315-349-6766     13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   135-349-6766     13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   REPORTABLE     CAUSE   SYSTEM   COMPONENT   FACTURER     NG   N/A   N   Image: System component   FACTURER     YES (If yes, complete 15. EXPECTED   NO   NO   DATE   DATE     ABSTRACT (Lim				20.2	0.2203(a)(2)(i) 50.36(c)			(1)(i)(A	l)	$\perp$	50.73(a)(2	2)(iii)	50.73(a)(2)(ix)(A)			ı)		
100   20.2203(a)(2)(iii)   50.36(c)(2)   50.73(a)(2)(v)(A)   73.71(a)(4)     20.2203(a)(2)(iv)   50.46(a)(3)(ii)   50.73(a)(2)(v)(B)   73.71(a)(4)     20.2203(a)(2)(iv)   50.46(a)(3)(ii)   50.73(a)(2)(v)(B)   73.71(a)(4)     20.2203(a)(2)(v)   50.73(a)(2)(v)(B)   73.71(a)(5)     20.2203(a)(2)(v)   50.73(a)(2)(i)(A)   50.73(a)(2)(v)(C)   OTHER     20.2203(a)(2)(vi)   50.73(a)(2)(i)(B)   50.73(a)(2)(v)(D)   Specify in Abstract below or in NRC Form 366A     ILICENSEE CONTACT     Mr. Chris M. Adner, Regulatory Assurance Manager   TELEPHONE NUMBER (Include Area Code)     315-349-6766   315-349-6766     ILICENSEE CONTACT FOR THIS LER     COMPONENT FAILURE POR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT     CAUSE SYSTEM COMPONENT FAILURE REPORTABLE TO EPIX     NANU- FACTURER     COMPONENT FAILURE COMPONENT FAILURE SYSTEM COMPONENT FAILURE TO EPIX     NG   N/A   N   N     14. SUPPLEMENTAL REPORT EXPECTED   NO   NO   NO     YES (If yes, complete 15. EXPECTED SUBMISSION DATE)   NO   NO   NO	10. POWER	≀ LEVEL		20.2203(a)(2)(ii) 50.3			50.36(c)	(1)(ii)(A	۹)	$\perp$	50.73(a)(2	?)(iv)(A)	50.73	3(a)(2)	)(x)			
100 20.2203(a)(2)(iv) 50.46(a)(3)(ii) 50.73(a)(2)(v)(B) 73.71(a)(5)   20.2203(a)(2)(v) 50.73(a)(2)(i)(A) 50.73(a)(2)(v)(C) OTHER   20.2203(a)(2)(vi) 50.73(a)(2)(i)(B) 50.73(a)(2)(v)(D) Specify in Abstract below or in NRC Form 366A   20.2203(a)(2)(vi) 50.73(a)(2)(i)(B) 50.73(a)(2)(v)(D) Specify in Abstract below or in NRC Form 366A   12. LICENSEE CONTACT FOR THIS LER   TELEPHONE NUMBER (Include Area Code) 315-349-6766   TACMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   CAUSE SYSTEM COMPONENT FACTURER REPORTABLE TO EPIX   CAUSE SYSTEM COMPONENT FACTURER REPORTABLE TO EPIX   NG N/A N/A N   14. SUPPLEMENTAL REPORT EXPECTED 15. EXPECTED SUBMISSION DATE) NO NOTH DAY YEAR   SUBMISSION DATE) NO   ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)   On September 22, 2015 at 17:03, with James A, FitzPatrick Nuclear Power Plant operating at 100 percent power, the				20.2	203(a)(2)(i	(iii)		50.36(c)	(2)			50.73(a)(2	?)(v)(A)	73.7	l(a)(4)	)		
Image: Construct of the second se	4	100		20.2	203(a)(2)(i	(iv)		50.46(a)	(3)(ii)		50.73(a)(2)(v)(B)			73.7	73.71(a)(5)			
Image: 20.2203(a)(2)(vi) 50.73(a)(2)(i)(B) 50.73(a)(2)(v)(D) Specify in Abstract below or in NRC Form 366A   12. LICENSEE CONTACT FOR THIS LER   LICENSEE CONTACT FOR THIS LER   TELEPHONE NUMBER (Include Area Code)   Mr. Chris M. Adner, Regulatory Assurance Manager   TELEPHONE NUMBER (Include Area Code)   13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   CAUSE   SYSTEM   COMPONENT   FACTURER REPORTABLE   TO EPIX CAUSE SYSTEM   COMPONENT FACTURER REPORTABLE   TO EPIX CAUSE SYSTEM COMPONENT   MANU- FACTURER REPORTABLE COMPONENT MANU-   TO EPIX CAUSE SYSTEM COMPONENT MANU-   In SUPPLEMENTAL REPORT EXPECTED NO Intervent of the submission DATE MONTH DAY   YES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO NO DATE MONTH DAY   ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) Dn Date Date		100		20.2203(a)(2)(v)			50.73(a)(2)(i)(A)			N)	✓ 50.73(a)(2)(v)(C)							
12. LICENSEE CONTACT FOR THIS LER   TELEPHONE NUMBER (Include Area Code) 315-349-6766   TELEPHONE NUMBER (Include Area Code) 315-349-6766   TELEPHONE NUMBER (Include Area Code) 315-349-6766   COMPOLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT   CAUSE SYSTEM COMPONENT REPORTABLE TO EPIX CAUSE SYSTEM COMPONENT REPORTABLE TO EPIX   NG N/A N IS. EXPECTED MONTH DAY YEAR   VES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO NO DATE   ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) On September 22, 2015 at 17:03, with James A. FitzPatrick Nuclear Power Plant operating at 100 percent power, the				20.2	203(a)(2)(	vi)		50.73(a)	(2)(i)(B	5)		50.73(a)(2	?)(v)(D)	Spec NRC	ify in Ab Form 36	stract b 56A	elow or in	
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Emergency and Plant Information Computer (EPIC) indicated a spike in Secondary Containment (SC) differential pressure (d/P) during performance of a surveillance test associated with automatic isolation of SC and initiation of the Standby Gas Treatment System. Per the plant data systems SC d/P exceeded the Technical Specification (TS) allowed value, and then immediately trended negative following auto-start of one of the trains of Standby Gas Treatment.

The time period that SC d/P was greater than the TS allowed value is reportable pursuant to 10 CFR 50.72(b)(3)(v)(C) and 10 CFR 50.73(a)(2)(v)(C), as an event or condition that could have prevented fulfillment of a safety function. SC was operable following reestablishment of greater than or equal to 0.25 inches of water vacuum, and remains operable.

SC d/P excursions during transition from normal to isolation mode of the Reactor Building Ventilation (RBV) System are an expected condition, and attributable to the design of the non-safety related RBV System. The cause of the SC d/P exceeding the TS allowed value has been determined not to be associated with a component failure or equipment malfunction. Similar reportable events were identified during preparation of this report. A comprehensive listing of these occurrences is included in the report.

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		required to resp	ond to, the inform	nation collection.			
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#### NARRATIVE

### Background

The Secondary Containment (SC) [EIIS identifier: NG] boundary surrounds the primary containment and refueling equipment. The boundary forms a control volume to contain, dilute, and hold up fission products. The SC consists of four systems which include the Reactor Building, the Reactor Building Isolation and Control System, the Standby Gas Treatment (SBGT) System [BH], and the Main Stack. Secondary Containment is designed to provide containment for postulated design basis accident scenarios: loss-of-coolant accident and refueling (fuel handling) accident. Since pressure may increase in Secondary Containment relative to the environmental pressure, support systems are required to maintain a differential pressure vacuum such that external atmosphere would leak into containment rather than fission products leak out.

The systems which maintain a differential pressure vacuum inside Secondary Containment include the normal Reactor Building Ventilation and Cooling (RBV) System [VA] (during normal plant operations) and the safety-related Standby Gas Treatment System for post-accident conditions.

Technical Specification (TS) Surveillance Requirement (SR) 3.6.4.1.1 requires the SC differential pressure to be maintained more negative than 0.25 inches of water vacuum relative to the atmosphere. Failure to meet this SR, except for momentary transients due to gusty wind conditions (TS Bases 3.6.4.1.1), results in the Secondary Containment Limiting Condition for Operation (LCO) not being met, and requires the Secondary Containment to be declared Inoperable.

## **Event Description**

On September 22, 2015 at 17:03, with James A. FitzPatrick Nuclear Power Plant operating at 100 percent power, the Emergency and Plant Information Computer (EPIC) EPIC A-3348 indicated a spike in SC differential pressure during performance of a surveillance test associated with automatic isolation of SC and initiation of the SBGT.

An operator was subsequently dispatched to the ventilation control panel. In general, the control room indications (i.e. the ventilation control panel) are used for operability determinations. They verified that Secondary Containment differential pressure was more negative than the Technical Specification allowed value; therefore, the SC was determined to remain Operable.

On September 29, 2015, as part of the investigation into RB differential pressure response during the process of isolating the reactor building, Operations determined that EPIC does correspond with plant status.

An NRC notification was made via ENS 51512 on 11/3/2015. This Licensee Event Report (LER) is being submitted per 10 CFR 50.73(a)(2)(v)(C) as an event or condition that could have prevented the fulfillment of a safety function to control the release of radioactive material. The failure to immediately report the condition was entered into the JAF Corrective Action Program.

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## LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

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# **Event Analysis**

### Cause

The Secondary Containment differential pressure has a tendency to move towards a slight positive, when the RBV is switched from normal to an isolation mode. This is also observed from the readings obtained in the control room for the SC pressure during the transition phase. The cause of the increase in d/P is the difference in closure time for the supply and exhaust isolation valves. The supply and exhaust isolation valves are designed to close within 15 and 5 seconds respectively. After the exhaust valves are closed (within the first 5 seconds), the operating supply fans keep bringing outside air in for the remaining 10 seconds of the supply valves closure, causing SC d/P to rise.

The condition of an increase in SC d/P during transition between normal and isolation modes of the RBV System is an expected response, and attributable to the design of the non-safety related RBV System. The cause of this condition is not associated with any component failure or malfunction.

# Similar Events

### **Internal Events**

The JAF Plant Data System (PDS) was utilized to review the Secondary Containment differential pressure response over the previous three years during surveillance testing resulting in isolation of the Secondary Containment and automatic initiation of the SBGT System (i.e. ST-34A and ST-34B). The review identified twelve (12) instances where the transition between normal and isolation mode of the RBV System resulted in a d/P that was less negative than the TS requirement. The results are shown in Table 1.

The JAF Paperless Condition Reporting System was reviewed to confirm that the Secondary Containment differential pressure responses depicted in Table 1 were not associated with any failure or component malfunction. The transitory spikes are an expected condition, and were not previously documented in the JAF Corrective Action Program.

Date		Surveillance Test	Peak	Duration		
			(in. wg)	(sec)		
	3/29/2013	ST-34B	-0.01	25		
ĺ	3/29/2013	ST-34B	+0.38	45		
	4/1/2013	ST-34B	-0.03	20		
	6/27/2013	ST-34A	+0.14	25		
	6/27/2013	ST-34A	-0.13	10		
	10/30/2014	ST-34B	+0.04	23		
	10/30/2014	ST-34B	-0.13	10		
	10/30/2014	ST-34B	-0.08	15		
	10/30/2014	ST-34B	-0.03	15		
	10/30/2014	ST-34B	-0.18	5		
	10/30/2014	ST-34B	-0.20	5		
ĺ	9/22/2015	ST-34A	+0.04	25		

#### Table 1 – Differential Pressure Response

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### External events:

Susquehanna Steam Electric Station; Unit 1: LER 2015-005, Loss of Secondary Containment Differential Pressure During Ventilation Damper Testing.

Columbia Generating Station; LER 2013-007-01, Secondary Containment Pressure Exceeded During Severe Weather Conditions.

Fermi Nuclear Generating Station; Unit 2: LER 2015-001-01, Secondary Containment Declared Inoperable After Loss of Reactor Building Ventilation from Freeze Protection Actuation.

# **Corrective Actions**

### **Completed Actions**

• Discuss the guidance in NUREG-1022 Revision 3 with applicable Operations and Regulatory Assurance personnel.

## **Safety Consequence and Implications**

There were no actual consequences caused by these events. SC provides a control volume to contain fission products that leak from primary containment, or are released directly to the secondary containment as a result of a Loss of Coolant Accident (LOCA) or Refueling Accident (RA).

The difference in closing time between the inlet and exhaust valves of the reactor building during transition from normal to isolate mode represents a potential exfiltration pathway for released activity. This potential exfiltration pathway has been conservatively quantified, and is included in the JAF design basis accident analyses. Dose consequence results remain well below the 10 CFR 100 and 10 CFR 50.67 guidelines for all postulated accident conditions.

The Design Basis (DB) LOCA event results in a rapid primary containment pressure increase and reactor water level decrease. Drywell High Pressure or Low Reactor Water Level signals directly isolate the RBV, and start SBGT. Fuel damage caused by a DB LOCA is not expected until a rise in fuel cladding temperature after coolant is lost. The SC differential pressure was positive for a maximum of approximately 45 seconds during the RBV isolations reported in this LER. Therefore, it would have occurred early in the DB LOCA event. RBV isolation and initiation of SBGT would have been completed prior to fuel failure and release of radiological materials.

The DB RA event results in a release of radioactive material by a dropped fuel assembly during refuel operations. Radiation detectors would detect the release and initiate SC isolation. The pressure changes reported in this LER could have caused some exfiltration before the isolation was complete; however, the amount of exfiltration, consequentially the offsite and control room doses, would remain below regulatory limits as analyzed.

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### References

- Condition Report: CR-JAF-2015-04198, September 22, 2015, Transitory Secondary Containment Differential Pressure Excursion
- Condition Report: CR-JAF-2015-04893, Failure to Immediately Report the Condition Documented in CR-JAF-2015-04198
- Technical Specifications
- TSTF-551, Revision 1, "Address Transient Secondary Containment Conditions"
- Nuclear Safety Evaluation: JAF-SE-96-071, Revision 2