

A 04/27/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
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50-269/270/287

REC: CASE E G
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DUKE PWR

DOC DATE: 04/21/78
DATE RCVD: 04/26/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 0

FURNISHING ADDL INFO RE APPLICANT'S LTR DTD 04/14/78 CONCERNING APPARENT
ERROR IN THE B&W SMALL BREAK ECCS ANALYSIS APPLICABLE TO SUBJECT FACILITY &
CORRECTIVE ACTIONS IMPLEMENTED. . . ADVISING APPLICANT CONSIDERS UNITS CAN BE
OPERATED AT FULL RATED PWR SAFELY

PLANT NAME: OCONEE - UNIT 1
OCONEE - UNIT 2
OCONEE - UNIT 3

REVIEWER INITIAL: XJM
DISTRIBUTER INITIAL: *M*

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:
1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LICENSE.
(DISTRIBUTION CODE A001)

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DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

April 21, 1978

US NRC
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Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. R. Reid, Chief
Operating Reactors Branch #4

Reference: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

REGULATORY DOCKET FILE COPY

My letter of April 14, 1978 addressed an apparent error in the B&W small break ECCS analysis applicable to Oconee Nuclear Station and corrective actions which had been implemented to assure continued safe operation of the Oconee units. Subsequently, several communications have occurred involving Duke, B&W, and the staff with regard to the results of additional small break ECCS analyses. Specifically, B&W evaluations performed during the week of April 16, 1978 indicated results more severe than those addressed by my letter of April 14, 1978. This letter is to summarize our current understanding with regard to this matter and to provide certain pertinent information to the staff.

Based on our communications with the Oconee NSSS and fuel vendor, B&W, it is our understanding and position that Oconee can continue to be operated at full rated power (2568 Mwt core power) safely and in full compliance with the provisions of 10CFR50.46. In order to assure this, however, certain prompt operator actions would be required following a postulated small break LOCA, assuming various worst case single failures. Specifically, the HPI pump B-C discharge header cross-over valves (HP-116 and HP-117) and the HPI injection line A engineered safeguards valve (HP-26) must begin opening within five minutes of actuation of the engineered safeguards system and must be fully open within an additional ten minutes. Procedures have been established to assure that this requirement is met. This, in turn, assures that adequate timely flow to the core is provided.

In addition to the amendment proposed by our letter of April 20, 1978, a facility operating license amendment will be submitted by May 1, 1978 to require the procedures necessary to implement the actions discussed

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Mr. Edson G. Case, Acting Director
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above. Also, as soon as the information is finalized and available, a submittal with regard to the B&W small break ECCS analysis will be provided, to include: Break spectrum information; identification of worst case break; justification of peak clad temperatures if core uncover occurs; and justification of any necessary operator actions. It is also our intent to submit within three months (i.e., by July 21, 1978), proposed corrective action for the particular incident of present concern which will place less reliance on a high state of operator readiness. (It is our intention to demonstrate that the time available for operator action is equivalent to that indicated in my letter of April 14, 1978 - i.e., 25 minutes.).

In conclusion, we continue to consider that the Oconee units can be operated at full rated power safely and in compliance with the provisions of 10CFR 50.46.

Very truly yours,

William O Parker Jr / WSC

William O. Parker, Jr.

DCH:ge