F 7/19/78

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50-269/270/287

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DOCTYPE: LETTER NOTARIZED: NO

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SUBJECT:
REQUEST FOR RELIEF FROM THE REQUIREMENTS OF ASME CODE SEC. XI.

PLANT NAME: OCONEE - UNIT 1
OCONEE - UNIT 2
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REVIEWER INITIAL: XRS
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REACTOR SAFETY BR**W/ENCL
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J. MCGOUGH**W/ENCL
NRC PDR**W/ENCL
OELD**LTR ONLY
CORE PERFORMANCE BR**W/ENCL
ENGINEERING BR**W/ENCL
PLANT SYSTEMS BR**W/ENCL
EFFLUENT TREAT SYS**W/ENCL

EXTERNAL:
LPDR'S
WALHALLA, SC**W/ENCL
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NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 40 ENCL 39
SIZE: 1P+11P

CONTROL NBR: 782000088

********** THE END **********

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

July 14, 1978

Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. R. Reid, Chief
Operating Reactors Branch #4

Reference: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

Recently, a review of the requirements of ASME Code Section XI was conducted with respect to systems installed at Oconee Nuclear Station with a result that several requests for relief are required.

Therefore, and pursuant to 10CFR 50, §50.55a, please find attached requests for relief from the requirements of ASME Code Section XI.

These attached requests supplement those submitted on April 11 and May 8, 1978. No license fee is provided with this submittal as a license fee covering this issue was provided in my letter of May 23, 1978.

Very truly yours,

William O. Parker, Jr.

RLG:scs
Attachments

A001
S/1

DUKE POWER COMPANY

Request For Relief From
Inservice Inspection Requirement
Oconee Nuclear Station
Unit 1

Item 1

Reference Code: ASME Boiler & Pressure Vessel Code, Section XI, 1974 Edition through Summer 1975 Addenda

I. Component for which exemption is requested:

a. Name and Identification Number:
High Pressure Injection System
Reactor Coolant Pump Seal Supply Line
System 51A subsystem 7, isometric drawing No. 19, Part 1 of 2: Welds No. 1K & 1L

The seal supply line is 1 1/2 in. nominal pipe size with wall thickness of 0.281 in. Material: Type 316 stainless steel.

b. Function:

The seal supply line provides high pressure water to the reactor coolant pump shaft seal. Welds 1K and 1L are circumferential butt welds in the seal supply line.

c. ASME Section III Code Class:

ASME Section III Code Class: Class 3 per USNRC Regulatory Guide 1.26, Revision 2.

d. Valve Category:

Not Applicable.

II. Reference Code Requirement that has been determined to be impractical:

Welds 1K and 1L are new welds installed during a station modification performed during the 1977 shutdown.

Pressure testing in accordance with 1WD-5200 and 1WD-2600 was not possible.

III. Basis for Requesting Relief

Design pressure for this system is 3050 psig. In order to meet the requirements of part II above, the test pressure must be 3355. The test pressure exceeds that pressure allowed with fuel in the vessel.

IV. Alternate Examination:

These welds were radiographed and found acceptable during the 1977 refueling shutdown.

V. Implementation Schedule:

The examination of IV above was performed as a preservice examination. Welds 1K and 1L will be included in the inservice inspection plan for Class 3 pressure testing. If pressure testing is still impractical when scheduled, additional relief will be requested.

FORM QR 27	REVISION

Ellenburg

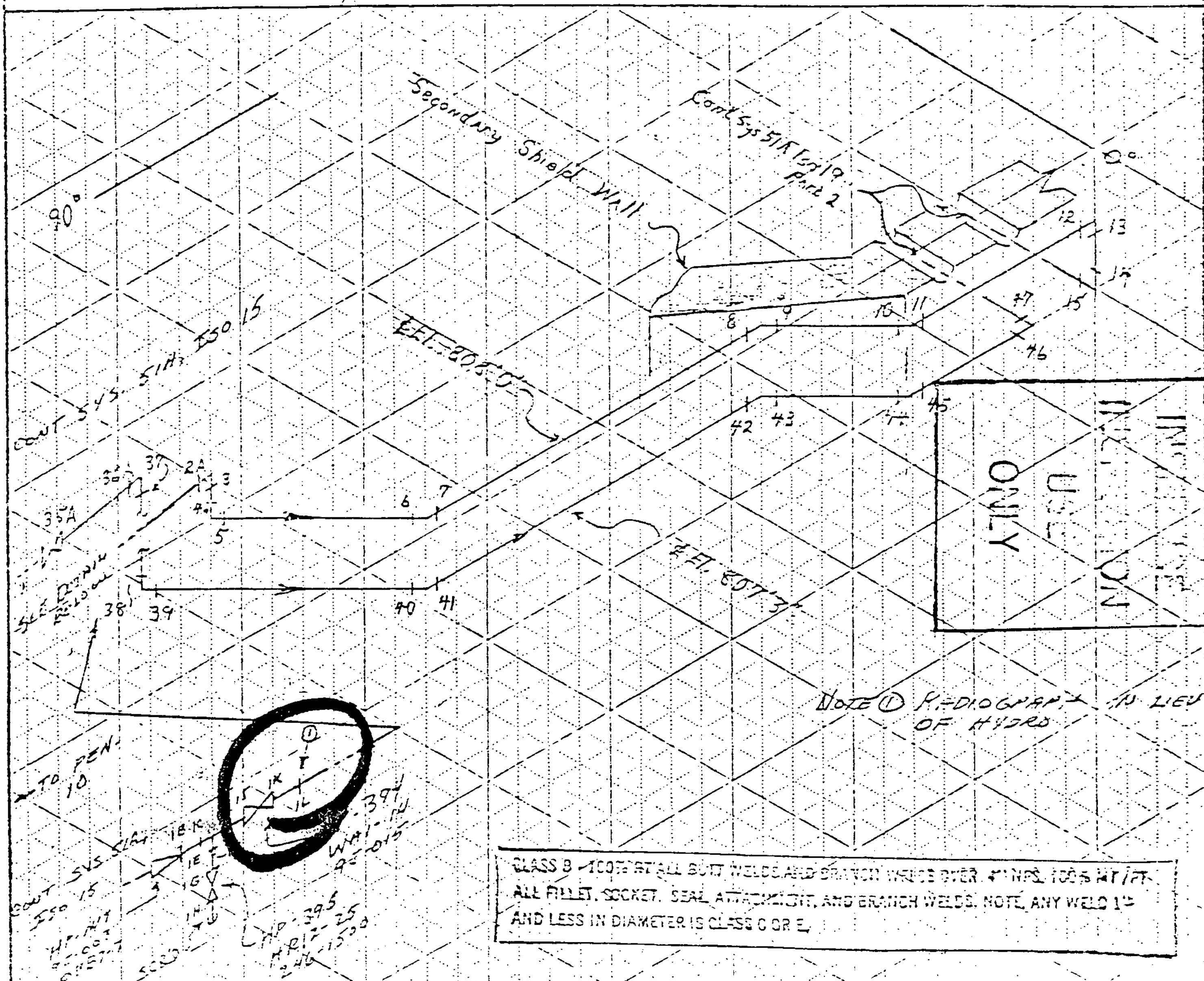
H.P. Supply to B Pumps

DUKE POWER COMPANY
CONSTRUCTION DEPARTMENT

ISOMETRIC SKETCH Part 1 of 2

PROJECT Oconee SYSTEM 51A SUB SYSTEMS (7) UNIT 1 R.B. ISO. NO.* 19 REV. NO. 9

CLASS B & C MATERIAL CRES/316 WELDING PROCEDURE L-201 L-204 P-7/2-B LAST WELD NO.* 39 DATE 9-16-77



REF. DWG. NOS. DWG.	REV.	SIZE × WALL THICKNESS	WELD NUMBERS	NDT CODE	ISO. REV. NO.	±	CHANGES WELD NOS.	ISO. REV. NO.	±	CHANGES WELD NOS.
479A	N/P	1½" x .281"	3-15, 37-47 [illegible]	6	6		Divided 2 Parts			
479D	1		2A, 36A, 1B-1E		7	+	1A 35A			
[illegible]-1018-1	A		1F-1L		7	-	1, 35			
					8	-	2, 36			
		½" ø x 0.188" 1E-1H			8	+	2A 36A			
					9		[illegible]			
					1		[illegible] 1E-1H 1F-1L			
[illegible]										

*ALL WELD NUMBERS SHOWN ABOVE ARE PRECEDED BY THE ISO. NO.

R.L.H. [illegible]

Item 2

Reference Code: ASME Boiler and Pressure Vessel Code, Section XI, 1974 Edition through Summer, 1975 Addenda

I. Component for which exemption is requested:

a. Name and Identification Number:
Low Pressure Injection and Core Flooding System (System 53)

1. Ref Drawings:
Diagrammatic Layout: PO-102A-1
Isometric Drawings: System 53A; isometric 2, part 2
System 53A; isometric 2, part 3

2. Weld Numbers: 46LA and 60 LB as indicated on referenced isometric drawing

b. Function

Welds 46 LA and 60LB are attachment welds to the process pipe of the Low Pressure Injection and Core Flooding System. 46LA is the attachment weld for a twin spring hanger; 60LB is the attachment for a rigid restraint.

c. ASME Section III Code Class:
Class 1 per USNRC Regulatory Guide 1.26, Revision 2.

d. Valve Category:
Not Applicable

II. Reference Code Requirement that has been determined to be impractical:

Table 1WB-2600, Item 4.9
Examination Category B-K-1

III. Basis for Requesting Relief

The weld geometry of welds 46LA and 60LB (Fillet) prevent a meaningful volumetric examination.

IV. Alternate Examination:

These welds were examined by the liquid dye penetrant method using techniques which met the requirements of the Reference Code. Examination was performed during the 1977 Refueling Shutdown.

V. Implementation Schedule:

These welds are scheduled for surface examination in accordance with the scheduling requirements of the Reference Code.

FORM QR 27 | REVISION 1

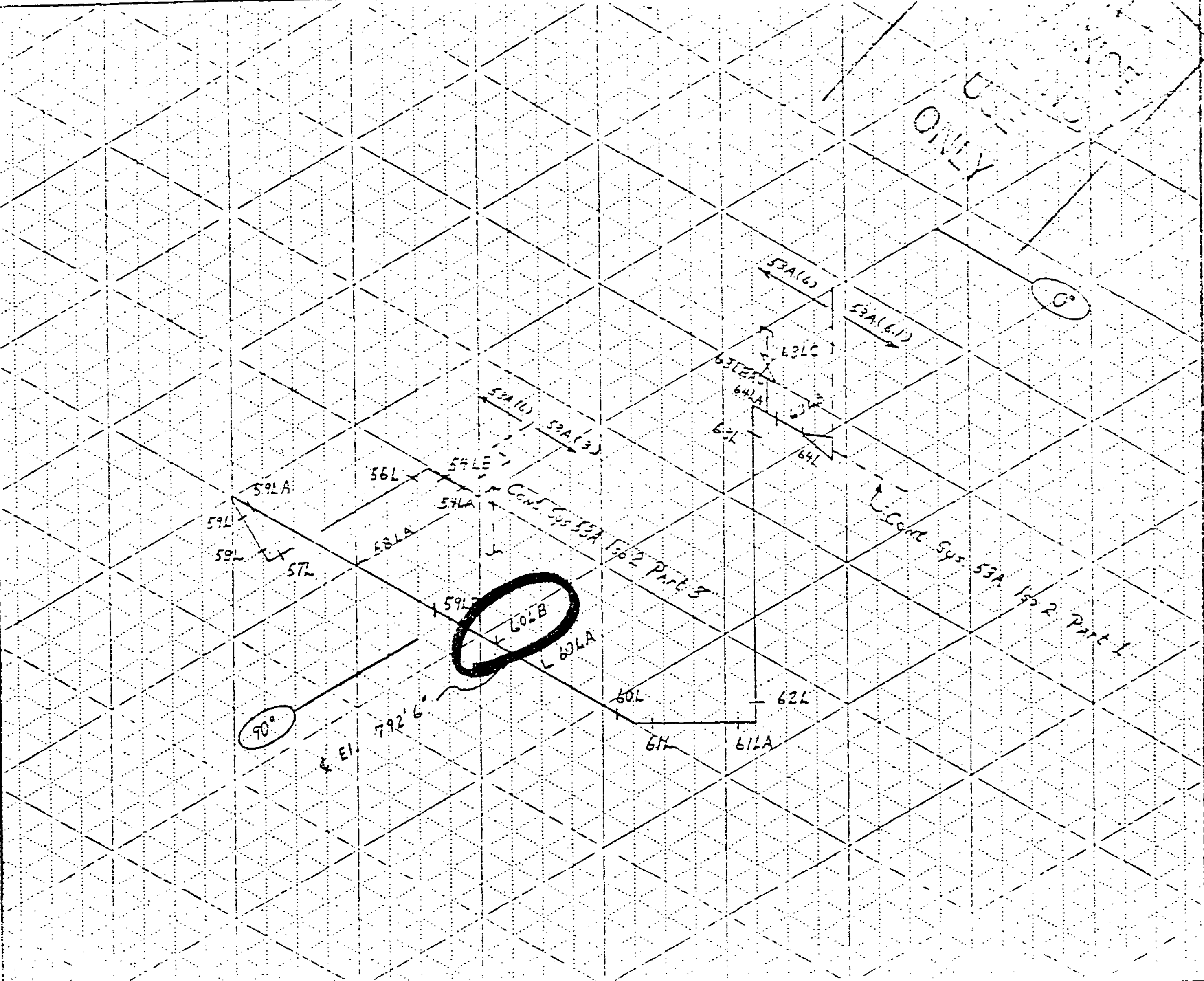
– Ellenburg

DUKE POWER COMPANY
CONSTRUCTION DEPARTMENT

ISOMETRIC SKETCH Part 2 of 4

PROJECT Oconee SYSTEM 53A SUB SYSTEMS 6 UNIT 1 ISO. NO.* 2 REV. NO. 14

CLASS A&C MATERIAL [illegible] WELDING PROCEDURE [illegible] LAST WELD NO.* 84 DATE [illegible]



REF. DWG. NOS. DWG.	REV.	SIZE × WALL THICKNESS	WELD NUMBERS	NDT CODE	ISC. REV. NO.	±	CHANGES WELD NOS.	ISC. REV. NO.	±	CHANGES WELD NOS.
478A		10" x [illegible]	[illegible]	1	7		[illegible]			
479B			[illegible]		13		[illegible]			
479D			[illegible]		11		[illegible]			
					14		[illegible]			
		[illegible]	[illegible]	C						
		[illegible]	[illegible]	6						

*ALL WELD NUMBERS SHOWN ABOVE ARE PRECEDED BY THE ISC. NO.

[illegible]

[illegible] 12 Dec 72

FORM QR 27 REVISION 1

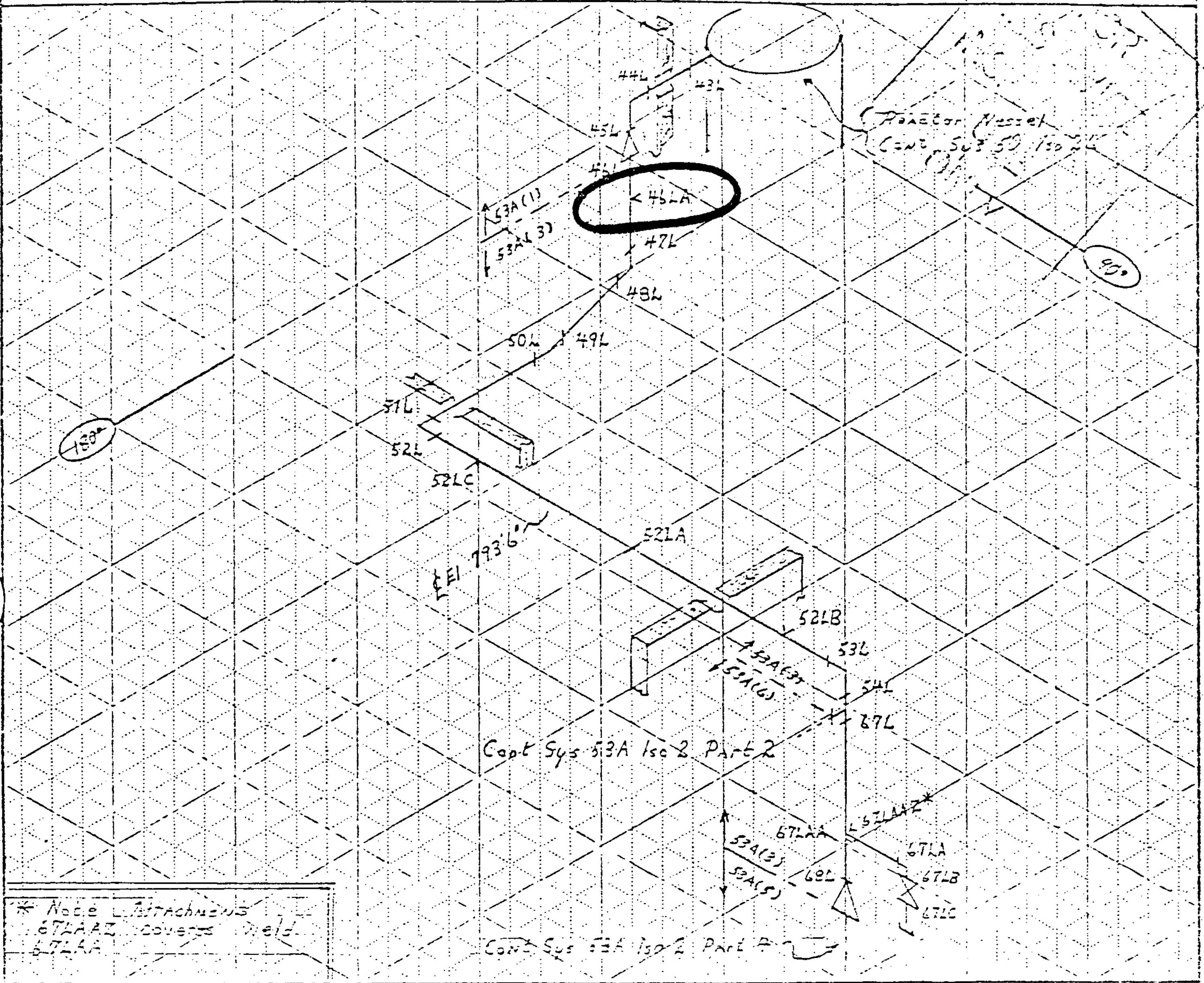
Ellenburg

DUKE POWER COMPANY
CONSTRUCTION DEPARTMENT

ISOMETRIC SKETCH *Part 3 of 4*

PROJECT *OCONEE* SYSTEM *53A* SUB SYSTEMS *(1)(3)* UNIT *1* ISO. NO.* *2* REV. NO. *14*

CLASS *A&C* MATERIAL *[illegible]* WELDING PROCEDURE *[illegible]* LAST WELD NO.* *84* DATE *[illegible]*



REF. DWG.* NOS. DWG.	REV.	SIZE × WALL THICKNESS	WELD NUMBERS	NDT CODE	ISO. REV. NO.	±	CHANGES WELD NOS.	ISO. REV. NO.	±	CHANGES WELD NOS.
#784		14"ø × 1.250"	43L-45L A [illegible]	1	14		Divided 4 Parts			
#78P		14"ø × 1.250"	46L-54L 52LA A [illegible]	1	14	[illegible]	Removed 46LA			
#78P			47L 49L		14		67L 67LAA 67LA			
					14		67LB 67LC			
					14		67LAAZ			
		1"ø × .250"	67LAA 67LA 67LB C [illegible]	6						
			67LC							
		Attachments	46LA 52LC 52LB 67LAAZ	C						

*ALL WELD NUMBERS SHOWN ABOVE ARE PRECEDED BY THE ISO. NO.

RLM

[illegible signature] 12 Dec 72

Item 3

Reference Code: ASME Boiler and Pressure Vessel Code Section XI 1970 Edition Including Winter 70 Addenda and 1974 Edition through Summer 1975 Addenda

I. Component for which exemption is requested:

a. Name and Identification Number:

Reactor Pressure Vessel
Clad Patch Examination

b. Function:

Reactor Core Support; Reactor Coolant Pressure Boundary

c. ASME Section III Code Class:

Equivalent Class I per NRC Regulatory Guide 1.26

d. Valve Category:

Not Applicable

II. Reference Code Requirement that has been determined to be impractical:

ASME Boiler and Pressure Vessel Code Section XI Portions as Follows:

1970 Edition including Winter 1970 Addenda Table IS-251 Category I-1

1974 Edition through Summer 1975 Addenda Table IWB-2500 Category B-I-1 and Paragraph IWB-2411.

III. Basis for Requesting Relief

The above portions of Section XI require that 25% of the reactor vessel clad patches be examined by the expiration of 40 months of commercial operation (and 50% by 80 months). Performance of these examinations requires complete defueling of the core and removal of the core barrel. This requirement is therefore, considered impractical.

IV. Alternate Examination:

The Code required NDE will be performed in accordance with item 5 below.

V. Implementation Schedule:

All clad patches will be examined at the end of the first 10-year interval.

DUKE POWER COMPANY

Request For Relief From
Inservice Inspection Requirement
Oconee Nuclear Station
Unit 2

Item 1

Reference Code: ASME Boiler and Pressure Vessel Code Section XI 1970 Edition Including Winter 70 Addenda and 1974 Edition through Summer 1975 Addenda

I. Component for which exemption is requested:

a. Name and Identification Number:

Reactor Pressure Vessel
Clad Patch Examination

b. Function:

Reactor Core Support; Reactor Coolant Pressure Boundary

c. ASME Section III Code Class:

Equivalent Class 1 per NRC Regulatory Guide 1.26

d. Valve Category:

Not Applicable

II. Reference Code Requirement that has been determined to be impractical:

ASME Boiler and Pressure Vessel Code Section XI Portions as Follows:

1970 Edition including Winter 1970 Addenda Table IS-251 Category I-1

1974 Edition through Summer 1975 Addenda Table IWB-2500 Category B-I-1 and Paragraph IWB-2411.

III. Basis for Requesting Relief

The above portions of Section XI require that 25% of the reactor vessel clad patches be examined by the expiration of 40 months of commercial operation (and 50% by 80 months). Performance of these examinations requires complete defueling of the core and removal of the core barrel. This requirement is therefore, considered impractical.

IV. Alternate Examination:

The Code required NDE will be performed in accordance with item 5 below.

V. Implementation Schedule:

All clad patches will be examined at the end of the first 10-year interval.

DUKE POWER COMPANY

Request For Relief From
Inservice Inspection Requirement
Oconee Nuclear Station
Unit 3

Item 1

Reference Code: ASME Boiler and Pressure Vessel Code Section XI 1970 Edition Including Winter 70 Addenda and 1974 Edition through Summer 1975 Addenda

I. Component for which exemption is requested:

a. Name and Identification Number:

Reactor Pressure Vessel
Clad Patch Examination

b. Function:

Reactor Core Support; Reactor Coolant Pressure Boundary

c. ASME Section III Code Class:

Equivalent Class I per NRC Regulatory Guide 1.26

d. Valve Category:

Not Applicable

II. Reference Code Requirement that has been determined to be impractical:

ASME Boiler and Pressure Vessel Code Section XI Portions as Follows:

1970 Edition includeing Winter 1970 Addenda Table IS-251 Category I-1

1974 Edition through Summer 1975 Addenda Table IWB-2500 Category B-I-1 and Paragraph IWB-2411.

III. Basis for Requesting Relief

The above portions of Section XI require that 25% of the reactor vessel clad patches be examined by the expiration of 40 months of commercial operation (and 50% by 80 months). Performance of these examinations requires complete defueling of the core and removal of the core barrel. This requirement is therefore, considered impractical.

III. Basis for Requesting Relief (cont.)

IV. Alternate Examination:

The Code required NDE will be performed in accordance with item 5 below.

V. Implementation Schedule:

All clad patches will be examined at the end of the first 10-year interval.