

A 09/27/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)  
DISTRIBUTION FOR INCOMING MATERIAL

50-287

REC: OREILLY J P  
NRC

ORG: PARKER W O  
DUKE PWR

DOC DATE: 09/19/78  
DATE RCVD: 09/26/78

DOCTYPE: LETTER NOTARIZED: NO  
SUBJECT:

COPIES RECEIVED  
LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-287/78-013) ON 08/23/78 CONCERNING  
DURING PWR OPERATION, THE TORQUE SWITCH FOR VALVE 31PSW-24 FAILED (DUE TO  
CORROSION) THUS CAUSING THE VALVE TO BE INOPERABLE IN THE CLAD POSITION...  
/ATT.

PLANT NAME: OCONEE - UNIT 3

REVIEWER INITIAL: XJM  
DISTRIBUTOR INITIAL: *ml*

\*\*\*\*\* DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS \*\*\*\*\*

NOTES:  
M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

INCIDENT REPORTS  
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF ORB#4 BC\*\*W/4 ENCL

INTERNAL: REG FILE\*\*W/ENCL  
I & E\*\*W/2 ENCL  
I & C SYSTEMS BR\*\*W/ENCL  
NOVAK/CHECK\*\*W/ENCL  
AD FOR ENO\*\*W/ENCL  
HANAUER\*\*W/ENCL  
AD FOR SYS & PROJ\*\*W/ENCL  
ENGINEERING BR\*\*W/ENCL  
KREGER/J. COLLINS\*\*W/ENCL  
K SEYFRIT/IE\*\*W/ENCL

NRC PDR\*\*W/ENCL  
MIPC\*\*W/3 ENCL  
EMERGENCY PLAN BR\*\*W/ENCL  
EEB\*\*W/ENCL  
PLANT SYSTEMS BR\*\*W/ENCL  
AD FOR PLANT SYSTEMS\*\*W/ENCL  
REACTOR SAFETY BR\*\*W/ENCL  
VOLLMER/BUNCH\*\*W/ENCL  
POWER SYS BR\*\*W/ENCL

EXTERNAL: LPDR'S  
WALHALLA, SC\*\*W/ENCL  
NSIC\*\*W/ENCL  
ACRS CAT B\*\*W/16 ENCL

*ADY*

DISTRIBUTION: LTR 44 ENCL 44  
SIZE: 1P+1P+1P

CONTROL NBR: 781770045

\*\*\*\*\* THE END \*\*\*\*\*

*[Handwritten signature]*

REGULATORY DOCKET FILE COPY

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

September 19, 1978

TELEPHONE: AREA 704  
373-4083

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

RE: Oconee Unit 3  
Docket No. 50-287

Dear Mr. O'Reilly:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-287/78-13.

Very truly yours,

*William O. Parker, Jr.*  
William O. Parker, Jr. *By [Signature]*

KRW:scs  
Attachment

cc: Director, Office of Management Information  
and Program Control

1978 SEP 24 2 45  
REGULATORY SERVICES  
COMMUNICATIONS UNIT

781770045

A002/s\*  
1/1

DUKE POWER COMPANY  
OCONEE UNIT 3

Report Number: RO-287/78-13

Report Date: September 19, 1978

Occurrence Date: August 23, 1978

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: Reactor Building Cooling Unit Inoperable

Conditions Prior to Occurrence: 100% FP

Description of Occurrence:

On August 22, 1978, the torque switch for 3LPSW-24 failed, causing the valve to be inoperable in the closed position (ES position is open). On August 23, 1978 during the performance of OP/3/A/1104/15 the position indicator for 3LPSW-21 indicated that the valve had failed in an intermediate position. The two apparent valve failures removed two of the three Reactor Building cooling units (RBCU) from service. A unit shutdown (10%/hr) was commenced at 0830 as required by Oconee Nuclear Station Technical Specification 3.3.1. At 0935 it was determined that 3LPSW-21 was in its full open position and that the apparent valve failure was actually a failure of the position indicator. The unit shutdown was terminated.

Apparent Cause of Occurrence:

The cause of the inoperability of 3LPSW-24 was the failure of the valve operator torque switch. The torque switch failed due to corrosion.

The cause of the 'apparent' inoperability of 3LPSW-21 was the failure of the valve's position indicator to properly show the valve's position. The cause of this failure was the maladjustment of the open/close limit switch.

Analysis of Occurrence:

The apparent inoperability of two-out-of-three RBCU's did not, in fact, occur. One train of the RB Cooling System was indeed inoperable due to the failure of 3LPSW-24 (discharge valve on RBCU 3C). The 3B train was declared inoperable due to the failure of the indicator to accurately reflect the position of 3LPSW-21 (discharge valve on RBCU 3C). The valve was functioning properly throughout the incident and would have assumed its ES position, if required. Thus only one train of the RB Cooling System was inoperable and its inoperability lasted less than 24 hours. Technical Specification 3.3.5 allows the inoperability of one train for periods up to 24 hours for repairs and maintenance. Therefore, there were no adverse effects on public health and safety as a result of this occurrence.

Corrective Action:

The torque switch on LPSW-24 was replaced and the limit switch on LPSW-21 was readjusted to properly reflect the valve's position.

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: \_\_\_\_\_ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | S | C | N | E | E | 3 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

CONT  
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 8 | 7 | 7 | 0 | 8 | 2 | 3 | 7 | 8 | 8 | 0 | 9 | 1 | 9 | 7 | 8 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | During power operation, the torque switch for valve 3LPSW-24 failed causing  
03 | the valve to be inoperable in the closed position. This caused one train of  
04 | RB cooling to be inoperable. The two redundant trains were available if  
05 | necessary. No adverse effects to the public resulted.

06 |  
07 |  
08 |

09 | SYSTEM CODE | S | B | 11 | CAUSE CODE | E | 12 | CAUSE SUBCODE | D | 13 | COMPONENT CODE | V | A | L | V | E | X | 14 | COMP. SUBCODE | F | 15 | VALVE SUBCODE | A | 16

17 | LER/RO REPORT NUMBER | 7 | 8 | 21 | EVENT YEAR | 7 | 8 | 22 | SEQUENTIAL REPORT NO. | 0 | 1 | 3 | 24 | OCCURRENCE CODE | 0 | 3 | 28 | REPORT TYPE | L | 30 | REVISION NO. | 0 | 32  
18 | ACTION TAKEN | A | 19 | FUTURE ACTION | Z | 20 | EFFECT ON PLANT | Z | 21 | SHUTDOWN METHOD | Z | 22 | HOURS | 0 | 0 | 0 | 37 | ATTACHMENT SUBMITTED | V | 23 | NRC-4 FORM SUB. | Y | 24 | PRIME COMP. SUPPLIER | L | 25 | COMPONENT MANUFACTURER | L | 2 | 0 | 0 | 28

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The valve's torque switch failed due to corrosion. The switch was replaced.  
11 |  
12 |  
13 |  
14 |

15 | FACILITY STATUS | E | 28 | % POWER | 1 | 0 | 0 | 29 | OTHER STATUS | N/A | 30 | METHOD OF DISCOVERY | A | 31 | DISCOVERY DESCRIPTION | Operator Observation | 32

16 | ACTIVITY CONTENT RELEASED OF RELEASE | Z | 33 | Z | 34 | AMOUNT OF ACTIVITY | N/A | 35 | LOCATION OF RELEASE | N/A | 38

17 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | N/A | 39

18 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | N/A | 41

19 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | N/A | 43

20 | PUBLICITY ISSUED | Z | 44 | DESCRIPTION | N/A | 45