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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS) DISTRIBUTION FOR INCOMING MATERIAL

· 50-287

REC: OREILLY J P NRC

ORG: PARKER W O DUKE PWR

DOCDATE: 04/19/78 DATE RCVD: 04/24/78

DOCTYPE: LETTER NOTARIZED: NO COPIES RECEIVED SUBJECT: LTR 1 ENCL 1 LICENSEE EVENT REPT. NO. RO 50-287/78-9 CONCERNING VALVE 3HP-24 INOPERABILITY.

PLANT NAME: OCONEE - UNIT 3

REVIEWER INITIAL: XRL DISTRIBUTOR INITIAL:

NOTES:

1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

INCIDENT REPORTS (DISTRIBUTION CODE A002)

FOR ACTION:

BR CHIEF REID**W/4 ENCL

INTERNAL:

REG FILE**W/ENCL 1-8-E**W72 ENCL SCHROEDER/IPPOLITO**W/ENCL NOVAK/CHECK**W/ENCL KNIGHT**W/ENCL HANAUER**W/ENCL EISENHUT**W/ENCL SHA0**W/ENCL KREGER/J. COLLINS**W/ENCL K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL MIPC**W/3 ENCL HOUSTON**W/ENCL EEB**W/ENCL BUTLER**W/ENCL TEDESCO**W/ENCL BAER**W/ENCL VOLLMER/BUNCH**W/ENCL ROSA**W/ENCL

EXTERNAL:

DISTRIBUTION:

SIZE: 1P+1P+1P

LPDR'S WALHALLA, SC**W/ENCL TIC**W/ENCL NSIC**W/ENCL ACRS CAT B**W/16 ENCL

ENCL 45

COPIES NOT SUBMITTED PER **REGULATORY GUIDE 10.1**

CONTROL NER:

781150044

LTR 45

THE END

DUKE POWER COMPANY

Power Building 422 South Church Street, Charlotte, N. C. 28242

WILLIAM O. PARKER, JR. VICE PRESIDENT STEAM PRODUCTION

TELEPHONE: AREA 704 373-4083

April 19, 1978

Mr. James P. O'Reilly, Director U. S. Nuclear Regulatory Commission Suite 1217 230 Peachtree Street, Northwest Atlanta, GA 30303



RE: Oconee Unit 3 Docket No. 50-287

Dear Mr. O'Reilly:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-287/78-9.

Very truly yours,

William O. Parker Jr. William O. Parker, Jr. by WAH

KRW/vmb

REGULATORY DORKET FILE COPY

Attachment



Director, Office of Management Information and Program Control

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A002/5 *

DUKE POWER COMPANY

OCONEE UNIT 3

<u>Report Number</u>: R0-287/78-9

Report Date: April 19, 1978

Occurrence Date: March 20, 1978

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: Valve 3HP-24 Inoperability

Conditions Prior to Occurrence: 100% Full Power

Description of Occurrence:

At 0924, on March 20, 1978, during the performance of PT/0/A/0310/12A &12B (monthly ES test), Valve 3HP-24 failed to operate on manual actuation from the control room. At 0945, during subsequent investigation of the incident, a second attempt was made to open the valve and it functioned properly. The contacts in the valve operator were cleaned and the valve was cycled to verify operability. At 1030, the valve was returned to service.

Apparent Cause of Occurrence:

The cause of the failure was foreign material on the contacts of the valve's operator.

Analysis of Occurrence:

Technical Specification 3.3.2(b) requires the operability of all Engineered safeguard values related to the HPI System, but Technical Specification 3.3.5 allows up to 24 hours to repair inoperable components (with some exceptions). Value 3HP-24 was inoperable for a maximum of 66 minutes and operated properly without any corrective action shortly subsequent to the initial failure to function. It is considered that this incident did not adversely effect public health and safety.

Corrective Actions:

Valve 3HP-24 was examined, repaired and returned to service.

NRC FO (7-77)	RM 355 U. S. NUCLEAR REGULATORY COMMISSION
.e	LICENSEE EVENT REPORT EXHIBIT A
	CONTROL BLOCK:
	S C N E E 3 2 0 0 - 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0
	REPORT L 6 0 5 0 0 0 2 8 7 0 0 3 2 0 7 8 8 0 4 1 9 7 8 9 SOURCE 50 61 DOCKET NUMBER 56 59 EVENT DATE 74 73 REPORT DATE 10
02	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (1) During performance test PT/0/A/0310/12A & 12B valve HP-24, which controls
03	flow from the BWST to the A&B HPI pumps failed to open when actuated by
04	manual pushbutton. It was examined, its operator's contacts were cleaned, and
05	it was returned to service in 66 minutes. No Technical Specification limits
06	were exceeded. It is considered that the event did not adversely effect
07	public health and safety.
	SYSTEM CAUSE CAUSE COMPONENT CODE SUBCODE STREAM CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE STREAM CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE SUBCODE 11 E 2 13 13 14 15 17 18 19 20 16
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10	The probable cause of the failure was "dirt" on the operator's contacts.
	The contacts were cleaned and the valve operated properly. No further
12	corrective action was required.
1]]	
14	
15	ACCIUTY & POWER OTHER STATUS 30 NETHOD OF DISCOVERY DESCRIPTION 32 E 39 11 0 0 30 NA 12 DUring monthly ES test.
	CTIVITY CONTENT 12 13 44 48 48 10 10 10 10 ACTIVITY (35) 10 LOCATION OF RELEASE (36)
7 8	PERSONNEL EXPOSUAES
17	NUMBER TYPE DESCRIPTION (39) 0 0 0 0 0 2 39 11 12 13
18	PERSONNEL INJURIES
7 8	
19	NA NA
20	PUBLICITY BO SSUED DESCRIPTION (45) NAC USE ONLY NA I I I I I I I I I I I I I I I I I I I
	9 10 63 69 80 NAME OF PREPARER K. R. Wilson 80 80 80
	PHONE: (704) 575-0197

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