

E 03/03/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-269/270/287

REC: OREILLY J P
NRC

ORG: PARKER W D
DUKE PWR

DOCDATE: 02/28/78
DATE RCVD: 03/02/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 1

LICENSEE EVENT REPT (RO 50-287/78-002/03L-1) ON 01/05/78 CONCERNING
AFTER SAMPLE WAS TAKEN FROM THE OTSG, FDW-108 FAILED TO
CLOSE. W/ATT LERS 269/78-001, 287/78-001, 270/77-017, 269/77-031
AND 270/78-002.

PLANT NAME: OCONEE - UNIT 1
OCONEE - UNIT 2
OCONEE - UNIT 3

REVIEWER INITIAL: XUM
DISTRIBUTER INITIAL:

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:
L. M. CUNNINGHAM - ALL AMENDMENTS TO PSAR AND CHANGES TO TECH SPECS

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF REID**W/4 ENCL

INTERNAL: REG FILE**W/ENCL
L. M. CUNNINGHAM**W/2 ENCL
SCHRQEDER/IPPOLITO**W/ENCL
NOVAK/CHECK**W/ENCL
KNIGHT**W/ENCL
HANAUER**W/ENCL
EISENHUT**W/ENCL
SHAO**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
HOUSTON**W/ENCL
GRIMES**W/ENCL
BUTLER**W/ENCL
TEDESCO**W/ENCL
BAER**W/ENCL
VOLLMER/BUNCH**W/ENCL
ROSA**W/ENCL

EXTERNAL: LPDR'S
WALHALLA, SC**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

COPIES NOT SUBMITTED PER
REGULATORY GUIDE 10.1

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+5P

CONTROL NBR: 780620045

***** THE END *****

DUKE POWER COMPANY

REGULATORY DOCKET FILE COPY

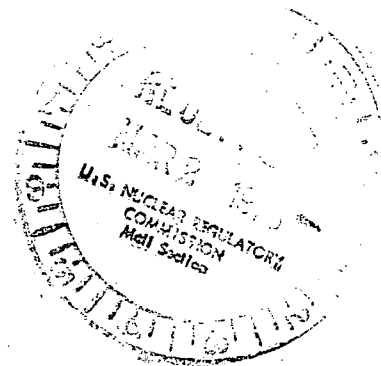
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28212

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

February 28, 1978

Mr. James P. O'Reilly
U. S. Nuclear Regulatory Commission
Suite 1217
230 Peachtree Street, Northwest
Atlanta, GA 30303



RE: Oconee Nuclear Station
Docket Nos. 269, 270, 287
Revision to Reportable Occurrence Reports

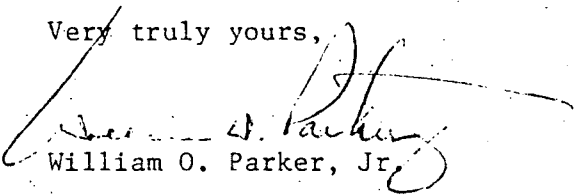
Dear Sir:

Please find attached revision to the Licensee Event Reports (LER) for Reportable Occurrence Reports:

RO - 287/78-2
RO - 269/78-1
RO - 287/78-1
RO - 270/77-17
RO - 269/77-31
RO - 270/78-2

submitted on January 26, February 3, and February 17, 1978. The revisions are transcriptional and typographical errors which were noticed on reviews subsequent to submittal of the originals.

Very truly yours,


William O. Parker, Jr.

KRW/rpc

Attachment

cc: Director, Office of Management Information
and Program Control

780620045

A002
5
1/1

PHONE: (704) 373-8197

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | S C N E | E | 1 | 2 | 0 0 | - | 0 | 0 0 | 0 | 0 - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO 57 CAT 58

0 1 | REPORT SOURCE | 1 | 5 | 0 | 5 | 0 | 0 | 0 | 2 | 6 | 9 | 7 | 0 | 1 | 0 | 5 | 7 | 8 | 8 | 0 | 2 | 2 | 8 | 7 | 8 | 9

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal operation Keowee Hydro Unit 2 failed to start on initiation

0 3 | from Oconee control room. Since Keowee 2 is a source of auxiliary power

0 4 | for the Oconee station (Units 1, 2, and 3) this is reportable under

0 5 | T.S. 6.6.2.1.b.(2). The unit started normally on second and subsequent

0 6 | attempts.

0 7 |

0 8 |

0 9 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17 | LER/RO REPORT NUMBER | 7 | 8 |

21 | 7 | 8 |

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24 | 0 | 0 | 1 |

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27 |

28 | 0 | 3 |

29 |

30 |

31 |

32 |

33 |

34 |

35 |

36 |

37 | 0 | 0 | 0 | 0 |

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41 | Y |

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43 | Y |

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W I I 2 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The Field Flashing Breaker (DB-26) on the generator failed to close on

1 1 | first attempt. The breaker was verified as operable by subsequent opera-

1 2 | tion and visual examination. Inspection and servicing resulted in no

1 3 | further corrective action.

1 4 |

1 5 | FACILITY STATUS | E | 28 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

% POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

1 6 | ACTIVITY CONTENT | Z | 33 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 7 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

TYPE DESCRIPTION (39)

1 8 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

DESCRIPTION (41)

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

DESCRIPTION (43)

2 0 | PUBLICITY ISSUED DESCRIPTION (45) | N | 44 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

DESCRIPTION (45)

NAME OF PREPARER K. R. Wilson

PHONE: (704) 373-8197

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 SICNEE 3200-000000-00034111145
7 8 9 14 15 25 26 57 CAT 58

CONT

01 REPORT SOURCE 60 61 050000287701037880228789
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 During performance of PT/0/A/203/5 Valve 3LP-14 failed to cycle manually.

03 It was repaired within one hour of the failure. The incident is reportable

04 under Section 6.6.2.1b(2) on the Tech Specs.

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SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
9 10 11 12 13 18 19 20
09 S B 11 E 12 B 13 V A L V E X 14 E 15 D 16
17 LER/RO REPORT NUMBER 21 22 23 24 25 26 27 28 29 30 31 32
01 7 8 0 0 1 0 3 L 1
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47
B 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 Y 24 L 25 F 1 2 7 26
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 When first trying to cycle 3LP-14 the valve moved slightly and the engaging

11 collar started to slip and became unaligned. The actuating push rod could

12 not be reconnected to the pneumatic control so the valve was inoperable

13 both manually and pneumatically. The collar was realigned and the setscrew

14 and handwheel were tightened.

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NAME OF PREPARER K. R. Wilson

PHONE: (704) 373-8197

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																								
01 S C N E E 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5					7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32																													
CONT 01					REPORT SOURCE L 8					DOCKET NUMBER 05010102697					EVENT DATE 122977					REPORT DATE 022878														
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10																																		
On December 27, an increase in reactor coolant leakage was noted. At 2131																																		
RPS Channel A tripped on press/temp. It was determined that a pressure switch																																		
had blown. Reactor shutdown was initiated but was terminated when switch was																																		
valved out. On December 29, it was noted that Channel 1 WR pressure was																																		
reading high so the channel was tripped. At 1545 recalibration was complete																																		
so channel was returned to service. At no time was the ability of the reactor																																		
to operate safely impaired.																																		
SYSTEM CODE 11 CAUSE CODE 12 CAUSE SUBCODE 13 COMPONENT CODE 14 COMP. SUBCODE 15 VALVE SUBCODE 16																																		
17 LER/RO REPORT NUMBER 77 18 EVENT YEAR 77 19 SEQUENTIAL REPORT NO. 031 20 OCCURRENCE CODE 03 21 REPORT TYPE L 22 REVISION NO. 1																																		
ACTION TAKEN 18 FUTURE ACTION 19 EFFECT ON PLANT 20 SHUTDOWN METHOD 21 HOURS 22 ATTACHMENT SUBMITTED 23 NPRO-4 FORM SUB. 24 PRIME COMP. SUPPLIER 25 COMPONENT MANUFACTURER 26																																		
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27																																		
The ES Analog Channel 1 WR pressure transmitter is located about five feet																																		
from and slightly below the blown pressure switch. It is likely that steam																																		
from the switch struck the transmitter thus throwing it out of calibration.																																		
Recalibration corrected the transmitter error. Valving out the switch has																																		
eliminated the leak. The transmitter was a model 56PH.																																		
FACILITY STATUS 28 % POWER 29 OTHER STATUS 30 METHOD OF DISCOVERY 31 DISCOVERY DESCRIPTION 32																																		
15 F 090 NA B During performance test PT/1&2/600/1																																		
ACTIVITY CONTENT 33 RELEASED OF RELEASE 34 AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36																																		
16 Z Z NA NA																																		
PERSONNEL EXPOSURES 37 TYPE 38 DESCRIPTION 39																																		
17 000 Z NA																																		
PERSONNEL INJURIES 40 DESCRIPTION 41																																		
18 000 NA																																		
LOSS OF OR DAMAGE TO FACILITY 42 TYPE 43 DESCRIPTION 44																																		
19 Z NA																																		
PUBLICITY 45 ISSUED 46 DESCRIPTION 47																																		
20 N NA																																		
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