

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)  
DISTRIBUTION FOR INCOMING MATERIAL 50-287

REC: OREILLY J P  
NRC

ORG: PARKER W O  
DUKE PWR

DOCDATE: 02/03/78  
DATE RCVD: 02/07/78

DOCTYPE: LETTER NOTARIZED: NO COPIES RECEIVED  
SUBJECT: LTR 1 ENCL 1  
LICENSEE EVENT REPORT R0-287/78-2 ON 1/4/78 CONCERNING FAILURE OF  
FDW-108 TO CLOSE FOLLOWING SAMPLE.

PLANT NAME: OCONEE - UNIT 3

REVIEWER INITIAL: XEF  
DISTRIBUTOR INITIAL:

\*\*\*\*\* DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS \*\*\*\*\*

NOTES:  
1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

INCIDENT REPORTS  
(DISTRIBUTION CODE A002)

FOR ACTION: BRANCH CHIEF SCHWENCER\*\*W/4 ENCL

INTERNAL:

REG FILE\*\*W/ENCL  
~~1-W-E\*\*W/2 ENCL~~  
SCHROEDER/IPPOLITO\*\*W/ENCL  
NOVAK/CHECK\*\*W/ENCL  
KNIGHT\*\*W/ENCL  
HANAUER\*\*W/ENCL  
EISENHUT\*\*W/ENCL  
SHAO\*\*W/ENCL  
KREGER/J. COLLINS\*\*W/ENCL  
L. CROCKER\*\*W/ENCL

NRC PDR\*\*W/ENCL  
MIPC\*\*W/3 ENCL  
HOUSTON\*\*W/ENCL  
GRIMES\*\*W/ENCL  
BUTLER\*\*W/ENCL  
TEDESCO\*\*W/ENCL  
BAER\*\*W/ENCL  
VOLLMER/BUNCH\*\*W/ENCL  
ROSA\*\*W/ENCL

EXTERNAL:

LPDR'S  
WALHALLA, SC\*\*W/ENCL  
TIC\*\*W/ENCL  
NSIC\*\*W/ENCL  
ACRS CAT B\*\*W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45  
SIZE: 1P+2P

CONTROL NBR: 780390003

\*\*\*\*\* THE END \*\*\*\*\*

004  
JR

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

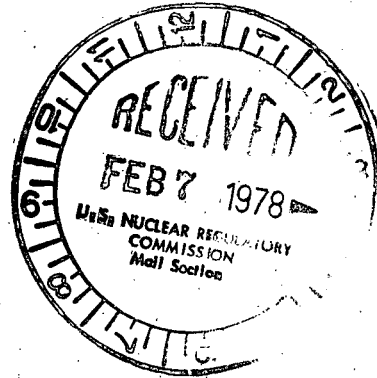
REGULATORY DOCKET FILE COPY

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

TELEPHONE: AREA 704  
373-4083

February 3, 1978

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Suite 1217  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303



RE: Oconee Unit 1  
Docket No. 50-287

Dear Mr. O'Reilly:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station  
Technical Specifications, please find attached Reportable Occurrence  
Report RO-287/78-2.

Very truly yours,

*William O. Parker Jr.*  
William O. Parker, Jr. *by WATT*

/mh

Attachment

cc: Director, Office of Management Information  
and Program Control

*A002/6  
0/1*

780390003

DUKE POWER COMPANY  
OCONEE UNIT 1

Report No.: RO-287/78-2

Report Date: February 3, 1978

Occurrence Date: January 4, 1978

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: FDW-108 failed to close following sample

Conditions Prior to Occurrence: 100 percent full power

Description of Occurrence:

On January 4 at 1040 after taking a sample from the OTSG, 3FDW-108 failed to close. Redundant valve FDW-107 was locked closed and its associated circuit breakers were locked open as required by Technical Specification 3.6.3b(2). After repacking, the valve was cycled several times without incident and was declared operable.

Apparent Cause of Occurrence:

This type of valve and identical valves have failed to operate several times previously. Corrective actions suggested by the manufacturer have been unsuccessful in stopping the failures. Evidently, the valves are not suitable for long-term operation in the system environment.

Analysis of Occurrence:

3FDW-108 is an isolation valve on the steam generator sample lines. The failure of the valve did not violate containment integrity as the redundant valve was operable and would have performed its Engineered Safeguard function if necessary. Thus, the health and safety of the public were not endangered.

Corrective Action:

Valve was repacked and valve was cycled to insure operability.

This failure is one of several similar failures which have been occurring at the Oconee Nuclear Station and no corrective actions have been successful thus far. Therefore, the valves (FDW 106, 108) will be replaced by others of suitable design at the first appropriate outage for each unit after September 1, 1978.

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: \_\_\_\_\_ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | S | C | N | E | E | 3 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5  
7 8 9 14 15 25 26 57 CAT 58

CON'T  
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 8 | 7 | 7 | 0 | 1 | 0 | 5 | 7 | 8 | 8 | 0 | 2 | 0 | 3 | 7 | 8 | 9  
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)  
02 | At 1040, after a sample was taken from the OTSG, FDW-108 failed to  
03 | close. It's redundant isolation valve FDW-107 operated properly so  
04 | containment integrity was maintained. The valve was repacked and  
05 | cycled to insure operability. Since containment integrity was not  
06 | jeopardized, the health and safety of the public were not endangered.  
07 |  
08 |

09 | SYSTEM CODE | H | J | 11 | CAUSE CODE | E | 12 | CAUSE SUBCODE | B | 13 | COMPONENT CODE | V | A | L | V | E | X | 14 | COMP. SUBCODE | F | 15 | VALVE SUBCODE | D | 16  
7 8 9 10 11 12 13 18 19 20  
17 | LER/RO REPORT NUMBER | 7 | 8 | 21 | 22 | SEQUENTIAL REPORT NO. | 0 | 0 | 1 | 24 | OCCURRENCE CODE | 0 | 3 | 28 | 29 | REPORT TYPE | L | 30 | REVISION NO. | 0 | 32  
18 | ACTION TAKEN | B | 18 | FUTURE ACTION | A | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | 37 | ATTACHMENT SUBMITTED | Y | 23 | NRC-4 FORM SUB. | Y | 24 | PRIME COMP. SUPPLIER | L | 25 | COMPONENT MANUFACTURER | R | 3 | 4 | 0 | 26  
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)  
10 | Failure of this type has occurred on numerous occasions, evidently  
11 | indications that this valve simply cannot withstand the necessary  
12 | environment. Therefore, the valves (FDW 106, 108) will be replaced  
13 | in the future by valves of appropriate design (see attachment).  
14 |

15 | FACILITY STATUS | E | 28 | % POWER | 1 | 0 | 0 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | B | 31 | DISCOVERY DESCRIPTION | Observation during test | 32  
7 8 9 10 12 13 44 45 46

16 | ACTIVITY CONTENT | Z | 33 | RELEASED OF RELEASE | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36  
7 8 9 10 11 44 45

17 | PERSONNEL EXPOSURES | NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39  
7 8 9 11 12 13

18 | PERSONNEL INJURIES | NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41  
7 8 9 11 12

19 | LOSS OF OR DAMAGE TO FACILITY | TYPE | Z | 42 | DESCRIPTION | NA | 43  
7 8 9 10

20 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | NA | 45  
7 8 9 10

NAME OF PREPARER: K. R. Wilson  
PHONE: 704/373-8197  
NRC USE ONLY

RECEIVED DOCUMENT  
CONTROL DESK

1976 FEB 7 PM 12 13

U.S. NRC  
DISTRIBUTION SERVICES  
BRANCH