

**NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)**

CONTROL NO: **7210**

FILE: INCIDENT REPORT FILE

FROM: Duke Power Company Charlotte, NC W O Parker Jr			DATE OF DOC 7-1-75	DATE REC'D 3-3-75	LTR XX	TWX	RPT	OTHER
TO: Mr Moseley			ORIG none signed	CC	OTHER	SENT AEC PDR <u>XX</u>		SENT LOCAL PDR <u>XX</u>
CLASS	UNCLASS XXXXXX	PROP INFO	INPUT	NO CYS REC'D 1		DOCKET NO: 50-287		

DESCRIPTION:

Ltr trans the following:

PLANT NAME: Oconee #3

ENCLOSURES:

Unusual Event # 75-7 on 5-23-75 concerning failure of valve 3FDW-108 to operate.....

**ACKNOWLEDGED  
DO NOT REMOVE**

FOR ACTION/INFORMATION

7-7-75

ehf

BUTLER (L) W/ Copies	SCHWENCER (L) W/ Copies	ZIEMANN (L) W/ Copies	REGAN (E) W/ Copies
CLARK (L) W/ Copies	STOLZ (L) W/ Copies	DICKER (E) W/ Copies	LEAR (L) W/ Copies
PARR (L) W/ Copies	VASSALLO (L) W/ Copies	KNIGHTON (E) W/ Copies	SPELS W/ Copies
KNIEL (L) W/ Copies	PURPLE (L) W/3Copies	YOUNGBLOOD (E) W/ Copies	W/ Copies

INTERNAL DISTRIBUTION

<del>REG FILE</del>	TECH REVIEW	DENTON	LIC ASST	A/T IND
<del>NRC PDR</del>	<del>SCHROEDER</del>	**GRIMES	R. DIGGS (L)	BRAITMAN
<del>OGC, ROOM P-506A</del>	<del>MACCARY</del>	<del>GAMMILL</del>	H. GEARIN (L)	SALTZMAN
<del>GOSSICK/STAFF</del>	<del>KNIGHT</del>	<del>KASTNER</del>	E. GOULBOURNE (L)	MELTZ
<del>CASE</del>	<del>PAWLICKI</del>	<del>BALLARD</del>	P. KREUTZER (E)	PLANS
<del>GIAMBUSSO</del>	<del>SHAO</del>	<del>SPANGLER</del>	J. LEE (L)	MCDONALD
<del>BOYD</del>	** <del>STELLO</del>	ENVIRO	M. MAIGRET (L)	CHAPMAN
<del>MOORE (L)</del>	** <del>HOUSTON</del>	MULLER	S. REED (E)	DUBE (Ltr)
<del>DEYOUNG (L)</del>	** <del>NOVAK</del>	DICKER	M. SERVICE (L)	E. COUPE
<del>SKOVHOLT (L)</del>	<del>ROSS</del>	KNIGHTON	S. SHEPPARD (L)	PETERSON
<del>GOLLER (L) (Ltr)</del>	<del>IPPOLITO</del>	YOUNGBLOOD	M. SLATER (E)	HARTFIELD (2)
<del>P. COLLINS</del>	<del>TEDESCO</del>	REGAN	H. SMITH (L)	<del>KLECKER</del>
<del>DENISE</del>	<del>J. COLLINS</del>	PROJECT LDR	S. TEETS (L)	EISENHUT
<del>REG OPR</del>	<del>LAINAS</del>	HARLESS	G. WILLIAMS (E)	WIGGINTON
<del>FILE &amp; REGION (2)</del>	<del>BENAROYA</del>		V. WILSON (L)	F. WILLIAMS
<del>MIPC/PE (3)</del>	<del>VOLLMER</del>		R. INGRAM (L)	HANAUER
<del>STEELE</del>				

EXTERNAL DISTRIBUTION

<del>1 - LOCAL PDR</del> <i>Walthalla, S.C.</i>	<del>1 - NATIONAL LABS</del>	1 - PDR-SAN/LA/NY
<del>1 - TIC (ABERNATHY) (1)(2)(10)</del>	<del>1 - W. PENNINGTON, Rm E-201 GT</del>	1 - BROOKHAVEN NAT LAB
<del>1 - NSIC (BUCHANAN)</del>	<del>1 - CONSULTANTS</del>	1 - G. ULRIKSON, ORNL
<del>1 - ASLB</del>	<del>NEWMARK/BLUME/AGBABIAN</del>	1 - AGMED (RUTH GUSSMAN) Rm B-127 GT
<del>1 - Newton Anderson</del>		1 - J. D. RUNKLES, Rm E-201 GT
<del>5 - ACRS SENT TO LIC ASST</del> <i>Sheppard</i>		
** SEND ONLY TEN DAY REPORTS		

DUKE POWER COMPANY

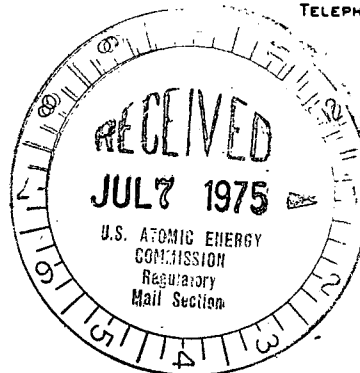
POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

TELEPHONE: AREA 704  
373-4083

July 1, 1975



Mr. Norman C. Moseley, Director  
U. S. Nuclear Regulatory Commission  
Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Re: Oconee Unit 3  
Docket No. 50-287

REGULATORY DOCKET FILE COPY//

Dear Mr. Moseley:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station  
Technical Specifications, please find attached Unusual Event  
Report UE-287/75-7.

Very truly yours,

*William O. Parker, Jr.*

William O. Parker, Jr. *by P.M.B.*

MST:vr  
Attachment

cc: Mr. Angelo Giambusso



DUKE POWER COMPANY  
OCONEE UNIT 3

Report No.: UE-287/75-7

Report Date: July 1, 1975

Event Date: May 23, 1975

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Event: Failure of valve 3FDW-108 to operate

Conditions Prior to Event: Unit at full power

Description of Event:

On May 23, 1975, the quarterly performance test of Engineered Safeguards valves was conducted on Oconee Unit 3. The B steam generator sample line isolation valve, 3FDW-108, failed to indicate closed after being given the close command. This valve is normally closed during operation and is opened only for steam generator sampling. The valve has been closed and locked until corrective maintenance can be performed.

Designation of Apparent Cause of Event:

The apparent cause of this event was a weak spring in the valve operator. The valve was found to be remotely operable with no pressure in the sample line; however, due to a weak spring in the operator, it would not close under pressure.

Analysis of Event:

Valve 3FDW-108 is the Reactor Building containment isolation valve for the B steam generator sample line. This valve is one of two redundant valves which are closed by an Engineered Safeguards signal in the unlikely event containment integrity is required. This incident did not affect the redundant isolation valve. It is concluded that the health and safety of the public was not affected by this incident.

Corrective Action:

A new spring has been ordered for the operator of valve 3FDW-108. Until the valve operator can be repaired, 3FDW-108 has been locked closed with the redundant isolation valve, 3FDW-107, also closed.

Failure Data:

A previous failure was experienced with valve 3FDW-108 and was discussed in report UE-287/74-4. At that time, no cause was found for the valve's failure to close in a specified time. After further testing, the valve was returned to service. The apparent cause of both failures has now been identified and will be corrected.