

50-287

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO: MR B C RUSCHE

FROM: DUKE POWER CO
CHARLOTTE, NC
W O PARKER, JR

DATE OF DOCUMENT
3-22-76

DATE RECEIVED
3-25-76

LETTER
 ORIGINAL
 COPY

NOTORIZED
 UNCLASSIFIED

PROP INPUT FORM

NUMBER OF COPIES RECEIVED
3 SIGN
22 COPY = 25

DESCRIPTION
LTR NOTORIZED 3-22- 6.....RE THEIR 3-16-76 LTR
REQUESTING PERMISSION TO REMOVE THE REACTOR
VESSELL SURVEILLANCE CAPSULES AND PUSH RODS
AT PRESENT TIME UNTIL END OF PRESENT FUEL
CYCLE... ..REQUEST APPROVAL NLT 4-2-76.....

ENCLOSURE

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: OCONEE 3

SAFETY

FOR ACTION/INFORMATION

ENVIRO

3-26-76 RB

ASSIGNED AD :		ASSIGNED AD :
BRANCH CHIEF : 6 PURPLE		BRANCH CHIEF :
PROJECT MANAGER:		PROJECT MANAGER :
LIC. ASST. : (17) SHEPPARD		LIC. ASST. :

INTERNAL DISTRIBUTION

REG FILE	SYSTEMS SAFETY	PLANT SYSTEMS	ENVIRO TECH
NRC PDR	HEINEMAN	TEDESCO	ERNST
I & E (2)	SCHROEDER	BENAROYA	BALLARD
OELD		LAINAS	SPANGLER
GOSSICK & STAFF	ENGINEERING	IPPOLITO	
MIPC	MACCARY		SITE TECH
CASE	KNIGHT	OPERATING REACTORS	GAMMILL
HANAUER	SIHWEIL	STELLO	STEPP
HARLESS	PAWLICKI		HULMAN
		OPERATING TECH	
PROJECT MANAGEMENT	REACTOR SAFETY	EISENHUT	SITE ANALYSIS
BOYD	ROSS	SHAO	VOLLMER
P. COLLINS	NOVAK	BAER	BUNCH
HOUSTON	ROSZTOCZY	SCHWENCER	J. COLLINS
PETERSON	CHECK	GRIMES	KREGER
MELTZ			
HELTAMES	AT & I	SITE SAFETY & ENVIRO	
SKOVHOLT	SALTZMAN	ANALYSIS	
	RUTBERG	DENTON & MULLER	

EXTERNAL DISTRIBUTION

LPRD. WALKHALA, SC	NATL LAB	BROOKHAVEN NATL LAB
TIC	REG. V-IE	ULRIKSON(ORNL)
NSIC	LA PDR	
ASLB	CONSULTANTS	
ACRS 16 [REDACTED] / SENT		

CONTROL NUMBER

3001

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

March 22, 1976

Mr. Benard C. Rusche
Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Oconee Unit 3
Docket No. 50-287

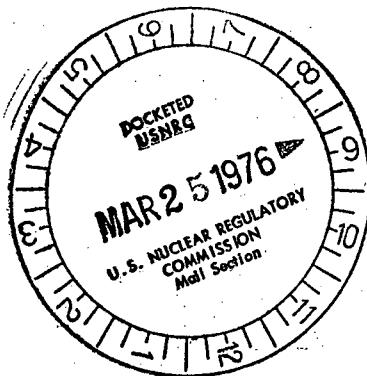
Dear Mr. Rusche:

My letter of March 16, 1976 reported that Duke Power Company had been appraised of damage to the reactor vessel surveillance holder tubes which had occurred in another Babcock and Wilcox reactor. A description was provided of an inspection of the Oconee 1 surveillance specimen tube holders, and it was stated that the following actions would be taken during Oconee 1, Cycle 3 operation:

1. The reactor vessel surveillance capsules and push rods will be removed.
2. The reactor vessel surveillance holder tubes will be secured from motion by a spring loaded retaining device which will be loaded into the upper end of each holder tube.

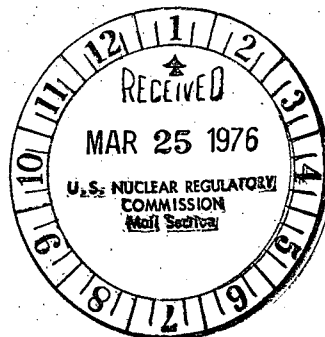
Requests for an exemption to the requirements of 10 CFR 50, Appendix H and a revision to the Oconee Nuclear Station Technical Specifications were filed to permit this action.

In an effort to minimize the possibility of wear damage to the Oconee 3 reactor vessel surveillance specimen holder tubes, the decision has been made to remove the reactor vessel surveillance capsules and push rods at the present time and, subsequently, to continue operation in a manner similar to Oconee 1 until the end of the present fuel cycle. An inspection of the surveillance holder tubes will be conducted, similar in extent to that performed on Oconee 1, and an evaluation will be performed to confirm their acceptability for operation for the remainder of the present fuel cycle.



Regulatory Docket File

TELEPHONE: AREA 704
373-4083



3001

Mr. Benard C. Rusche

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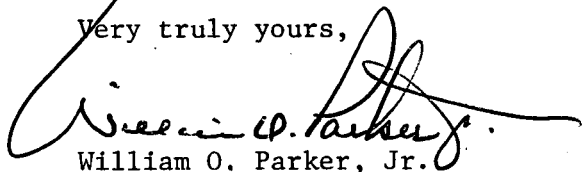
March 22, 1976

Oconee 3, Cycle 1 has currently accumulated 0.96 full power years of exposure for an equivalent capsule irradiation of 2.29 effective full power years, since the capsule receives 2.4 times the exposure of the inside $\frac{1}{4}$ wall thickness ($\frac{1}{4}t$) location of the reactor vessel beltline. Since available Oconee 3, Cycle 1 reactivity will allow operation for a maximum of approximately 486 EFPD (1.33 EFPY), there is ample margin between the present capsule irradiation of 2.29 EFPY and the maximum achievable exposure at the $\frac{1}{4}t$ reactor vessel beltline irradiation at the end of Cycle 1. In accordance with Technical Specification 4.2.9, a type B capsule removed from Oconee 3 during the present outage will be analyzed as part of the Reactor Vessel Surveillance Program.

Pursuant to 10 CFR 50, §50.12, an exemption from the requirements of 10 CFR 50, Appendix H, Section II.C.2 is requested to permit the continued operation of Oconee 3, Cycle 1 with the reactor vessel surveillance specimens removed from the reactor vessel. It is also requested, pursuant to 10 CFR 50, §50.90, that the Oconee Nuclear Station Technical Specifications be revised to indicate that the specimens will be removed from the reactor vessel for the remainder of Cycle 1 operation and that the surveillance withdrawal schedule will be revised prior to Cycle 2 operation. The attached proposed Technical Specification replacement page shows this change. It is considered that the operation, for the duration of this fuel cycle, with the reactor vessel surveillance specimens removed will not be inimical to the health and safety of the public.

Oconee 3 is scheduled to resume operation on April 5, 1976 following removal of the surveillance capsules. It is requested that this request be approved no later than April 2, 1976.

Very truly yours,



William O. Parker, Jr.

MST:mmb

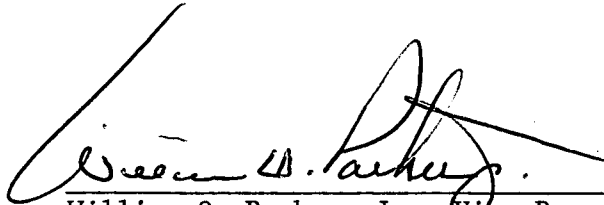
Attachment

Mr. Benard C. Rusche

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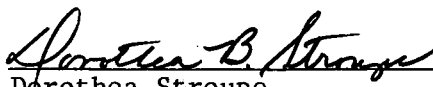
March 22, 1976

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47 and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.




William O. Parker, Jr., Vice President

ATTEST



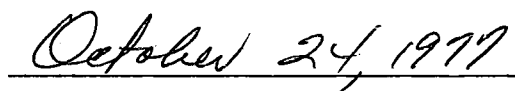
Dorothea Stroupe
Assistant Secretary

Subscribed and sworn to before me this 22nd day of March 1976.



Edna B. Larmey
Notary Public

My Commission Expires:



October 24, 1977

4.2.10

For Unit 1, the provisions of Specification 4.2.9 will be revised prior to Cycle 4 operation due to operation during Cycle 3 with the reactor vessel surveillance specimens removed from the reactor vessel. For Unit 3, the provisions of Specification 4.2.9 will be revised prior to Cycle 2 operation due to operation during a portion of Cycle 1 with the reactor vessel surveillance specimens removed from the reactor vessel.

4.2.11

During the first two refueling periods, two reactor coolant system piping elbows shall be ultrasonically inspected along their longitudinal welds (4 inches beyond each side) for clad bonding and for cracks in both the clad and base metal. The elbows to be inspected are identified in B&W Report 1364 dated December 1970.

Bases

The surveillance program has been developed to comply with Section XI of the ASME Boiler and Pressure Vessel Code, Inservice Inspection of Nuclear Reactor Coolant Systems, 1970, including 1970 winter addenda, edition. The program places major emphasis on the area of highest stress concentrations and on areas where fast neutron irradiation might be sufficient to change material properties.

The reactor vessel specimen surveillance program for Unit 1 and Unit 2 is based on equivalent exposure times of 1.8, 19.8, 30.6 and 39.6 years. The contents of the different type of capsules are defined below.

<u>A Type</u>	<u>B Type</u>
Weld Material	HAZ Material
HAZ Material	Baseline Material
Baseline Material	

For Unit 3, the Reactor Vessel Surveillance Program is based on equivalent exposure times of 1.8, 13.3, 26.7, and 30.0 years. The specimens have been selected and fabricated as specified in ASTM-E-185-72.

Early inspection of Reactor Coolant System piping elbows is considered desirable in order to reconfirm the integrity of the carbon steel base metal when explosively clad with sensitized stainless steel. If no degradation is observed during the two annual inspections, surveillance requirements will revert to Section XI of the ASME Boiler and Pressure Vessel Code.