

FILE:

FROM: Duke Power Company Charlotte, NC A. C. Thies	DATE OF DOC 8-5-74	DATE REC'D 8-7-74	LTR X	TWX	RPT	OTHER
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TO: L. Manning Muntzing	ORIG	CC 40	OTHER	SENT AEC PDR XXXX	SENT LOCAL PDR XXXX
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CLASS	UNCLASS XXXX	PROP INFO	INPUT XXX	NO CYS REC'D 40	DOCKET NO: 50-269/270/287
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DESCRIPTION:

Ltr notarized 8-5-74 submitting Appendix "K" Calculations & proposed Change to Tech Specs re 10CFR50, Appendix "K".....

** Denotes Letter Only

PLANT NAME: OCONEE UNITS 1, 2 & 3

ENCLOSURES: **ACKNOWLEDGED**

Amendment to the OL consisting of:

- 1) Loss of Coolant Accident Analysis conformance with 10CFR50.46, Appendix "K"
- 2) Proposed Change to tech specs...

DO NOT REMOVE

(40 cys encl rec'd)

FOR ACTION/INFORMATION 8-7-74 GMC

BUTLER (L) W/ CYS	SCHWENCER (L) W/ CYS	ZIEMANN (L) W/ CYS	REGAN (E) W/ CYS
CLARK (L) W/ CYS	STOLZ (L) W/ CYS	DICKER (E) W/ CYS	W/ CYS
PARR (L) W/ CYS	VASSALLO (L) W/ CYS	KNIGHTON (E) W/ CYS	W/ CYS
KNIEL (L) W/ CYS	PURPLE (L) W/9 CYS	YOUNGBLOOD (E) W/ CYS	W/ CYS

INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> REG FILE + 2 Ltrs	<u>TECH REVIEW</u>	DENYON	<u>LIC ASST</u>	A/T IND
<input checked="" type="checkbox"/> AEC PDR + 2 Ltrs	HENDRIE	GRIMES	DIGGS (L)	BRAITMAN
<input checked="" type="checkbox"/> OGC	SCHROEDER	GAMMILL	GEARIN (L)	SALTZMAN
<input checked="" type="checkbox"/> MUNTZING/STAFF	MACCARY	KASTNER	GOULBOURNE (L)	B. HURT
<input checked="" type="checkbox"/> **CASE	KNIGHT	BALLARD	KREUTZER (E)	
<input checked="" type="checkbox"/> **GIAMBUSSO	PAWLICKI	SPANGLER	LEE (L)	<u>PLANS</u>
<input checked="" type="checkbox"/> **BOYD	SHAO		MAIGRET (L)	MCDONALD
MOORE (L)(LWR-2)	<input checked="" type="checkbox"/> STELLO(2)	<u>ENVIRO</u>	REED (E)	CHAPMAN
DEYOUNG (L)(LWR-1)	HOUSTON	MULLER	SERVICE (L)	<input checked="" type="checkbox"/> **DUBE w/input
SKOVHOLT (L)	NOVAK	DICKER	<input checked="" type="checkbox"/> ** SHEPPARD (L)	<input checked="" type="checkbox"/> **E. COUPE
GOLLER (L)	ROSS	KNIGHTON	SLATER (E)	
P. COLLINS	IPPOLITO	YOUNGBLOOD	SMITH (L)	D. THOMPSON (2)
DENISE	TEDESCO	REGAN	TEETS (L)	KLECKER
<u>REG OPR</u>	LONG	PROJECT MGR	WILLIAMS (E)	EISENHUT
<input checked="" type="checkbox"/> FILE & REGION (2)	LAINAS	<input checked="" type="checkbox"/> SCALETTI	WILSON (L)	
MORRIS	BENAROYA	HARLESS		<input checked="" type="checkbox"/> VARGA
STEELE	VOLLMER			

EXTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> 1 - LOCAL PDR WALHALLA, SC	(1)(2)(10)-NATIONAL LABS	1-PDR-SAN/LA/NY
<input checked="" type="checkbox"/> 1 - TIC (ABERNATHY)	1-ASLBP(E/W Bldg, Rm 529)	1-BROOKHAVEN NAT LAB
<input checked="" type="checkbox"/> 1 - NSIC (BUCHANAN)	1-W. PENNINGTON, Rm E-201 GT	1-G. ULRIKSON, ORNL
1 - ASLB	1-B&M SWINEBROAD, Rm E-201 GT	1-AGMED (RUTH GUSSMAN) Rm B-127 GT
1 - P. R. DAVIS	1-CONSULTANTS	1-RD..MUELLER, Rm F-107 GT
<input checked="" type="checkbox"/> 16 - ACRS HOLDING	NEWARK/BLUME/AGBABIAN	
SENT TO LIC ASST SHEPPARD 8-774		

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2176

REGULATORY DOCKET FILE COPY

August 5, 1974

Mr. L. Manning Muntzing
Director of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545

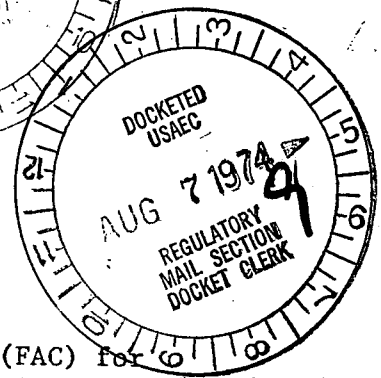
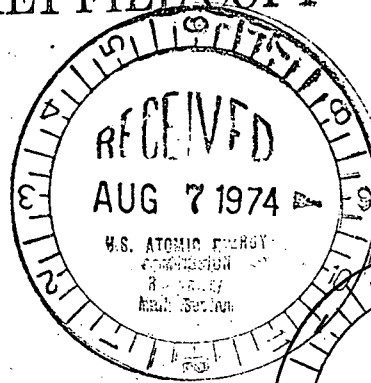
Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Muntzing:

On January 4, 1974 the AEC issued the Final Acceptance Criteria (FAC) for Emergency Core Cooling System (ECCS) evaluation as revisions to 10 CFR 50. In Compliance with these revisions, Babcock & Wilcox has developed an evaluation model which meets the requirements of Appendix K of 10 CFR 50. The description of this model is contained within their non-proprietary topical report BAW-10091, "B&W's ECCS Evaluation Model Report with Specific Application to 177 FA Class Plants with Lowered Loop Arrangement," which has been submitted to the Directorate of Licensing on August 5, 1974. In addition, B&W has provided supporting documentation for the computer codes utilized in this model in the following non-proprietary topical reports:

1. BAW-10092, "CRAFT 2- Fortran Program for Digital Simulation of a Multinode Reactor Plant During Loss of Coolant."
2. BAW-10093, "REFLOOD - Description of Model for Multinode Core Reflood Analysis."
3. BAW-10094, Babcock & Wilcox Revisions to THETA-B, a Computer Code for Nuclear Reactor Core Thermal Analysis - IN-1445.
4. BAW-10095, Babcock & Wilcox Revisions to CONTEMPT - Computer Program for Predicting Containment Pressure - Temperature Response to a Loss-of-Coolant Accident.

The analysis presented in BAW-10091 for the B&W 177 FA plants with lowered loop is generic in nature, since the plant parameters utilized in the analysis (such as the rated power level, fuel densification and containment building volume) are taken to be the most conservative values for all the plants of this type. Thus, the results contained in BAW-10091 provide an overly conservative analysis for all plants of this type and can be applied to Oconee Units 1, 2, & 3. As



Mr. L. Manning Muntzing

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such we adopt the results contained in BAW-10091. These results demonstrate conformance to the criteria of 10 CFR 50.46 under the following operating conditions:

1. The peak linear heat rate is less than or equal to 17.2kW/ft at the six foot elevation.
2. The Oconee Units 2 & 3 are operated within the attached revised Technical Specifications for the loss-of-coolant limits. These revisions were established on the basis of the LOCA limits as established from the FAC ECCS analysis (BAW-10091).
3. Oconee Unit 1 which has achieved in excess of 200 effective full power days of operation, can meet the final acceptance criteria as published in 10 CFR 50.46 without any changes to the existing technical specifications. Figure 1 shows the LOCA limit curve and the maximum operating peaks which are allowed under present technical specifications. Since in all cases the operating peaks, using existing technical specification limits, are below the LOCA limit the Final Acceptance Criteria are met.

Revisions to the rod withdrawal limits contained in the Oconee Nuclear Station Technical Specifications will be implemented on August 5, 1974. Continued operation of Oconee Units 1, 2, and 3 at rated power will be in compliance with 10 CFR 50.46. Technical specifications will be developed specifically for Oconee Units 2 and 3 using the methods developed in BAW-10091, and submitted to the AEC for review at a later date.

Very truly yours,

s/A. C. Thies
A. C. Thies

ACT:ch