NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

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DOCKET NUMBER: 50-269/270/287

FILE NUMBER:

TO:	FROM:	DATE OF DOCUMENT
Mr. Benard C. Rusche	Duke Power Company Charlotte, North Caroline Mr. William O. Parker, Jr.	7/30/76 DATE RECEIVED 8/6/76

LETTER	NOTORIZED	PROP	INPUT FORM	NUMBER OF COPIES RECEIVED
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DESCRIPTION

Ltr. notorized 7/30/76 w/attached..... requesting amdt. to ol/change to Appendix A tech spec.....furnishing replacement pages for tech spec 3.9.7.

(4-P)

ENCLOSURE

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME:

Oconee 1-2-3

SAFETY — FOR ACTION/INFORMATION — ENVIRO 8/6/76 RJL

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NRC FORM 195 (2-76)

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

July 30, 1976

Mr. Benard C. Rusche
Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Rusche:

Pursuant to 10CFR50, §50.90, an amendment to Section 3.9 of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, -47, -55 is requested. Proposed replacement pages for Technical Specification 3.9.7 are attached.

The proposed change to Technical Specification 3.9.7 provides provisions for implementing alternate measures to assure that release limits for liquid waste effluents are not exceeded whenever liquid waste monitors cannot be set to properly alarm and control liquid releases. This provision is considered necessary while efforts are being made to resolve problems with high background which have been experienced in the liquid effluent monitors. These high background readings prevent the setting of these monitors to function as required by Technical Specification 3.9.7.

The alternate proposed measures as stated in the specification change consist of redundant valve line-up checks and redundant sampling/analyses as discussed in our letter of May 14, 1976 submitted as a supplemental response to OIE Inspection Report 50-269/76-2. Currently, a task force is investigating the high background problem associated with the liquid effluent monitors. It is anticipated that this problem will be resolved by December 1, 1976 at which time the additional proposed requirements of Technical Specification 3.9.7 will be deleted.

In the interim, although no accurate alarm setpoint can be determined, liquid effluent monitors RIA-33 and -34 will be utilized to the extent possible to detect liquid effluent release rates.

Very truly yours,

William O. Parker, Jr.

7933

EDB:vr
Attachment

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47 and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

William O. Parker, Jr., Vice President

ATTEST:

John C. Goodman, Jr.
Assistant Secretary

Subscribed and sworn to before me this 30th day of July, 1976.

Edna B. Farmer
Notary Public

My Commission Expires:

October 24, 1977

3.9.3 The rate of release of radioactive materials in liquid waste from the station shall be controlled such that the instantaneous concentrations of radioactivity in liquid waste upon release from the Restricted Area, does not exceed the values listed in 10CFR20, Appendix B, Table II, Column 2.

3.9.4 The equipment installed in the liquid radioactive waste system shall be maintained and operated for the purposes of keeping releases within the objectives of these specifications and shall process all liquids prior to their discharge in order to limit the activity, excluding tritium and dissolved noble gases, released during any calendar quarter to 1.25 curies or less per unit.

3.9.5 As far as practicable, the releases of liquid waste shall be coordinated with the operation of the Keowee Hydro unit.

3.9.6 Liquid waste discharged from the liquid waste disposal system shall be continuously monitored during release. The liquid effluent monitor reading shall be compared with the expected reading of each discharge batch. The monitor shall be tested daily or prior to releases and calibrated at refueling intervals. The calibration procedure shall consist of exposing the detector to a referenced calibration source in a controlled, reproductible geometry. The sources and geometry shall be referenced to the original monitor calibration which provides the applicable calibration curves.

3.9.7 The effluent control monitor shall be set to alarm and automatically close the waste discharge valve such that the appropriate requirements of the specification are met.

In the event that the effluent control monitor is inoperable or cannot be calibrated to perform this function, the following will be performed to assure that prescribed release limits are not exceeded: A redundant valve lineup check of the effluent pathway and redundant sample analyses will be performed prior to each liquid effluent release.

These additional actions will be applicable until December 1, 1976.

3.9.8 In addition to the continuous monitoring requirements, liquid radioactive waste sampling and activity analysis shall be performed in accordance with Table 4.1.3. Records shall be maintained and reports of the sampling and analysis shall be submitted in accordance with Section 6.6 of these Technical Specifications.

Bases

It is expected that the releases of radioactive materials and liquid wastes will be kept within the design objective levels and will not exceed the concentration limits specified in 10CFR20. These levels provide the reasonable assurance that the resulting annual exposure to the whole body or any individual body organ will not exceed 5 millirem per year. At the same time, the licensee is permitted the flexibility of operation compatible with

considerations of health and safety to assure that the public is provided a dependable source of power under unusual operating conditions which may temporarily result in releases higher than design objective levels but still within the concentration limits specified in 10CFR20. It is expected that when using this operational flexibility under unusual operating conditions, the licensee shall exert every effort to keep the levels of radioactive materials and liquid wastes as low as practicable and that annual releases will not exceed a small fraction of the annual average concentration limits specified in 10CFR20.