

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

50-269/270/28  
FILE NUMBER

TO: N. R. C.		FROM: Duke Power Company Charlotte, North Carolina William O. Parker, Jr.		DATE OF DOCUMENT 5/6/77
LETTER <input checked="" type="checkbox"/> ORIGINAL <input type="checkbox"/> COPY		NOTORIZED <input checked="" type="checkbox"/> UNCLASSIFIED		DATE RECEIVED 5/13/77
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DESCRIPTION

Ltr. trans the following:

PLANT NAME: (2-P)  
Oconee Units 1-2-3

RJL

ENCLOSURE

Amdt. to OL/change to tech specs...notorized 5/6/77...concerns extension of the surveillance interval re hydraulic shock suppressors.

**ACKNOWLEDGED**

**DO NOT REMOVE**

(2-P)

SAFETY		FOR ACTION/INFORMATION		ENVIRO
ASSIGNED AD:		ASSIGNED AD:		
BRANCH CHIEF:	<i>Schwencer(S)</i>	BRANCH CHIEF:		
PROJECT MANAGER:	<i>Neighboys</i>	PROJECT MANAGER:		
LIC. ASST. :	<i>Sheppard</i>	LIC. ASST. :		

INTERNAL DISTRIBUTION			
<input checked="" type="checkbox"/> REG. FILE	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY
<input checked="" type="checkbox"/> NRC PDR	HEINEMAN	TEDESCO	ENVIRO ANALYSIS
<input checked="" type="checkbox"/> L. & E. (2)	SCHROEDER	BENAROYA	DENTON & MULLER
<input checked="" type="checkbox"/> OELD	ENGINEERING	LAINAS	AS
<input checked="" type="checkbox"/> GOSSICK & STAFF	MACARRY	IPPOLITO	ENVIRO TECH.
MIPC	BOSNAK	KIRKWOOD	ERNST
CASE	SIHWEIL	OPERATING REACTORS	BALLARD
<input checked="" type="checkbox"/> HANAUER	PAWLICKI	STELLO	YOUNGBLOOD
<input checked="" type="checkbox"/> HARLESS			
PROJECT MANAGEMENT	REACTOR SAFETY	OPERATING TECH.	SITE TECH.
BOYD	ROSS	EISENHUT	GAMMILL
P. COLLINS	NOVAK	SHAO	STEPP
HOUSTON	ROSZTOCZY	BAER	HULMAN
PETERSON	CHECK	BUTLER	SITE ANALYSIS
MELTZ		GRIMES	VOLLMER
HELTEMES	AT & I		BUNCH
SKOVHOLT	SALTZMAN		<input checked="" type="checkbox"/> J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION			CONTROL NUMBER
<input checked="" type="checkbox"/> LPDR: <i>Walthalla, SC</i>	NAT. LAB:	BROOKHAVEN NAT. LAB.	771360034 <i>AP 2</i> <i>GD</i>
<input checked="" type="checkbox"/> TIC:	REG V. IE	ULRIKSON (ORNL)	
<input checked="" type="checkbox"/> NSIC:	LA PDR		
ASLB:	CONSULTANTS:		
<input checked="" type="checkbox"/> ACRS / 6 CYS HOLDING/SENT	<i>As CAT B</i>		

# DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

TELEPHONE: AREA 704  
373-4083

May 6, 1977

Regulatory Docket File



Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Re: Oconee Nuclear Station  
Docket Numbers 50-269, -270, -287

Dear Sir:

Amendment Nos. 33, 33 and 30, to Oconee Facility Operating Licenses DPR-38, -47 and -55 were issued on October 13, 1976. These amendments provided Technical Specification Limiting Conditions for operation on hydraulic shock suppressors, as well as revised surveillance requirements. Specification 4.18.4 required that the initial inspection be performed within six months from the date of issuance of the Specifications, and for the purpose of establishing an inspection interval, it was assumed that the facility had been on a six month inspection interval.

The initial Oconee 2 inspection was performed on December 20, 1976, during a unit outage. During this inspection, 4 suppressors in the "inaccessible during normal operation" group were determined to be inoperable; therefore, the next inspection interval was determined to be 4 months + 25%. Since the initial inspection, Oconee 2 has been operating essentially continuously and no further inspections have been conducted. Currently, the last date permitted by Technical Specification for hydraulic shock suppressor surveillance is May 22, 1977.

Currently, with the expected operation of Oconee 1 and 2, both of these units could have overlapping refueling schedules. This is not considered advantageous from either a system load nor a station workload consideration. It is presently planned to refuel Oconee 1 first and Oconee 2 second. In order to accomplish this, a short outage will be conducted on Oconee 2 in late May, 1977, to perform a hydraulic shock suppressor surveillance as well as several other maintenance items.

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However, in order to provide for the contingency of an unexpected Oconee 1 forced outage, plans are also being made for a shortened Oconee 2 cycle which will permit the Oconee 2 refueling to be conducted first. If this contingency should occur, the outage for hydraulic shock suppressor surveillance would be inappropriate due to the time necessary to provide access to containment and to cool down the plant to permit personnel access.

Pursuant to 10CFR50, Section 50.90, a revision to the Oconee Nuclear Station Technical Specifications is requested which will revise the requirements of Specification 4.18 for Oconee 2 until June 19, 1977. This proposed revision is indicated on the attached Technical Specification replacement page. It is considered that this relatively short extension of the surveillance interval will not adversely affect the operation of the piping support and restraint system, and will not affect the health and safety of the public.

Very truly yours,

s/William O. Parker, Jr.  
William O. Parker, Jr.

MST:ge

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

s/William O. Parker, Jr.

\_\_\_\_\_  
William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 6th day of May, 1977.

s/Edna B. Farmer

\_\_\_\_\_  
Notary Public

My Commission Expires:

October 24, 1977  
\_\_\_\_\_

Applicability

Applies to hydraulic shock suppressors used to protect the Reactor Coolant System or other safety-related systems.

Objective

To verify that required hydraulic shock suppressors are operable.

Specification

4.18.1 All hydraulic snubbers listed in Table 4.18-1 whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected. This inspection shall include as a minimum hydraulic fluid reservoir, fluid connections, and linkage connections to the piping and anchor to verify suppressor operability in accordance with the following schedule:

<u>Number of Suppressors Found Inoperable During Last Inspection</u>	<u>Next Required Inspection Interval</u>
0	18 months <u>+</u> 25%
1	12 months <u>+</u> 25%
2	6 months <u>+</u> 25%
3,4	4 months <u>+</u> 25%
5,6,7	2 months <u>+</u> 25%
<u>&gt;8</u>	1 month <u>+</u> 25%

Note: (1) The required inspection interval shall not be lengthened more than one step per inspection.

Note: (2) Suppressors may be categorized in two groups, "accessible" or "inaccessible," based on their accessibility during reactor operation. These two groups may be inspected independently according to the above schedule.

Note: (3) The provisions of this specification shall not be applicable for Oconee 2 until after June 19, 1977.

4.18.2 All hydraulic snubbers whose seal materials are other than ethylene propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability once every month.

4.18.3 A representative sample of 10 hydraulic shock suppressors or approximately 10 percent of the suppressors installed, whichever is less, shall be functionally tested for operability each refueling outage. This test shall include verification of proper piston movement, lockup and bleed. For each suppressor determined to be inoperable, an additional 10 percent or 10 suppressors, whichever is less, shall be tested until no more failures are found or all suppressors have been tested. Suppressors with a rated capacity greater than 50,000 lbs. are exempted from this requirement.

4.18.4 The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule of Specification 4.18.1, it shall be assumed that the facilities had been on a 6 month inspection interval.