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FROM: Duke Power Company Charlotte, NC W O Parker		DATE OF DOC 6-27-75	DATE REC'D 7-1-75	LTR XXX	TWX	RPT	OTHER
TO: Mr Giambusso		ORIG one signed	CC	OTHER	SENT NRC PDR <u>XX</u>		SENT LOCAL PDR <u>XX</u>
CLASS	UNCLASS XXXXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-269/270/287		

DESCRIPTION:  
Ltr re our 2-19-75 ltr.....furnishing info concerning analyses & justification for a tech specs change regarding maintenance of containment hatches & isolation valves while containment integrity is required...

PLANT NAME: Oconee 1-3

ENCLOSURES:

**DO NOT REMOVE**

**ACKNOWLEDGED**

FOR ACTION/INFORMATION 7-3-75 ehf

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## DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

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373-4083

June 27, 1975

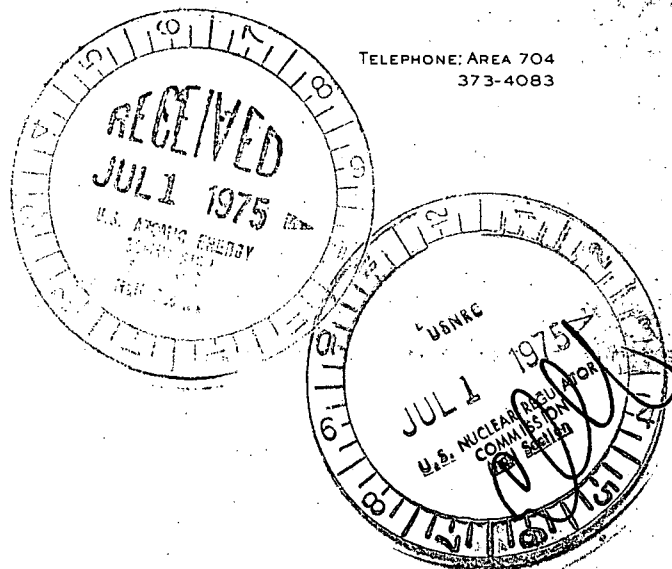
Mr. Angelo Giambusso, Director  
Division of Reactor Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Re: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287

Dear Mr. Giambusso:

Our letter of September 4, 1974 requested certain changes to Oconee Nuclear Station Technical Specification 3.6, "Reactor Building," which made provisions for maintenance of containment hatches and isolation valves while containment integrity is required. Subsequent to the approval of that request, it was determined that in some cases personnel passage through the operable door of an inoperable hatch is required in order to perform maintenance or test of the inoperable door. On January 15, 1975, a second change to Technical Specification 3.6 was requested which would permit this momentary passage. Mr. R. A. Purple's letter of February 19, 1975 responded to that request and stated that the proposed specification would permit containment integrity to be breached while at power operation. In addition, it stated that it appeared that alternative means of personnel access are available that would not involve a breach of containment integrity. It was requested that we reevaluate our proposal with a view toward furnishing additional analyses and justification to assure no undue risk to the public health and safety. The purpose of this letter is to provide these analyses and justification.

The hatch leak rate tests are performed by pressurizing the hatch between the inner (Reactor Building side) and outer (Auxiliary Building side) doors. This test tends to seat the outer door and unseat the inner door. Therefore, in order to perform the test, a restraint or strongback is installed on the hatch side of the inner door to keep the inner door seated. If the leak rate test fails due to the inner door gasket, it is impossible to enter the hatch from the Reactor Building side because the strongback is on the opposite side of the door. To repair the inner door gasket, momentary passage through the outer door is required.




Mr. Angelo Giambusso

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The hatch leak rate tests are performed quarterly and the inner hatch gasket does not fail frequently during this test. Further, the probability of the necessity for containment integrity during the momentary passage through the outer door is even more remote. In addition, the inner door would be shut during this time period and, even with a failed gasket, would limit the containment leakage in the unlikely event that containment integrity would be required. It is, therefore, concluded that the specification change requested in our January 15, 1975 letter will not create an undue risk to the public health and safety and it is requested that this Technical Specification change be approved as submitted.

Very truly yours,



William O. Parker, Jr.

MST:vr