

**FROM:**  
 Duke Power Company  
 Charlotte, N.C. 28201  
 A.C. Thies

**DATE OF DOCUMENT**  
 Mar. 8, 1972

**DATE RECEIVED**  
 Mar. 13, 1972

**NO:**  
 1284

**LTR.**  **MEMO:**  **REPORT:**  **OTHER:**

**TO:**  
 Mr. R.C. DeYoung

**ORIG.:**  **CC:**  **OTHER:**   
 3 signed

**ACTION NECESSARY**  **CONCURRENCE**  **DATE ANSWERED:**  
**NO ACTION NECESSARY**  **COMMENT**  **BY:**

**CLASSIF.:** U **POST OFFICE**  
 REG. NO:

**FILE CODE:**  
 50-269 50-270 50-287

**DESCRIPTION: (Must Be Unclassified)**  
 Ltr re our 2-10-72 ltr..furnishing addl  
 info on Inservice Inspection Program for  
 Ocone Station.....

REFERRED TO	DATE	RECEIVED BY	DATE
Schwencer w/4 cys for ACTION	3-14-72		

**ENCLOSURES:**

**DISTRIBUTION:**  
 Reg File Cy (3)  
 AEC PDR (3)  
 Compliance (2)  
 OGC-Rm-P-506-A  
 DeYoung  
 Case/Maccary  
 Hanauer  
 Skovholt

DO NOT REMOVE

**REMARKS:**  
 1 CY LOCAL PDR WALHALLA, S.C.

P. Collins  
 Schroeder  
 T.R. Wilson  
 Denton  
 Klecker

ACKNOWLEDGED

1284

DL

## DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES  
SENIOR VICE PRESIDENT  
PRODUCTION AND TRANSMISSION

March 8, 1972

United States Atomic Energy Commission  
Division of Reactor Licensing  
7920 Norfolk Avenue  
Bethesda, Maryland

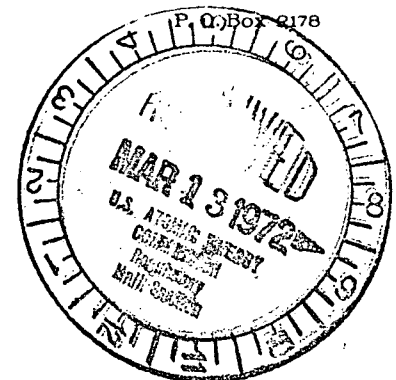
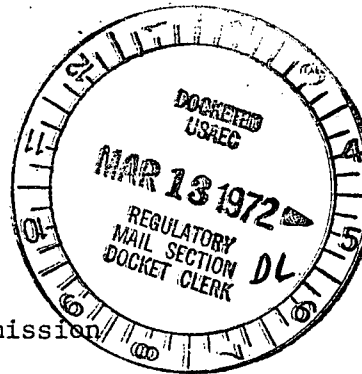
Attention: Mr. R. C. DeYoung, Assistant Director  
for Pressurized Water Reactors

Subject: Oconee Nuclear Station  
Docket Nos. 50-269, -270, and -287

Gentlemen:

Please reference your letter of February 10, 1972. The inspections that will be performed on the reactor coolant system at Oconee do comply with the intent of the "Recommended PWR Inservice Inspection Program for Detection of Effects of Reactor Coolant Leakage." The methods normally employed to detect coolant leakage are described in Section 3.1.6 of the Oconee Technical Specifications and provide an indication of the potential for prolonged contact of borated fluid with ferritic steel. Continuous surveillance of coolant inventory, activity monitoring, sump level monitoring, and physical inspection by operating personnel will identify coolant leakage during normal operation. A physical inspection of the reactor coolant system during each refueling shutdown will identify boric acid crystalline deposits from minute leakage during operation. Sources of leakage shall be corrected and ferritic steel components of the reactor coolant pressure boundary exposed to boric acid residue will be examined for evidence of corrosion. Prior to startup following each refueling outage, the reactor coolant system will be examined under not less than normal operating pressure to ensure leak tight integrity during operation as described in Section 4.3 of the Oconee Technical Specifications.

The program outlined above complies with our interpretation of the AEC's "Recommended PWR Inservice Inspection Program for Detection of Effects of Reactor Coolant Leakage." It is our interpretation that paragraph B(1)a of the recommended program describes corrective measures applied to leakage that cannot be eliminated and does not require provisions for collection and disposal of potential leakage as preventative measures.



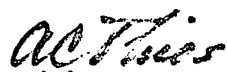
United States Atomic Energy Commission

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With the above clarification, we will comply with the requirements of the program recommended as a part of the Oconee Inservice Inspection Program.

Sincerely,

A handwritten signature in cursive script that reads "A. C. Thies".

A. C. Thies

ACT:vr