

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO: Mr. Edson G. Case

FROM: Duke Power Co.
Charlotte, N. C. 28242
William O. Parker

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07-11-77
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07-15-77

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DESCRIPTION
Ltr. Concluding that the current Tech Specs limits for Oconee Unit 1, 2, & 3 and the proposed Tech Specs changes for Oconee 1 Cycle 4 and Oconee 2 Cycle 3 are conservatively valid...

ACKNOWLEDGED

1 pages **DO NOT REMOVE**

PLANT NAME: OCONEE UNITS 1, 2, & 3
jcm 07/18/77

ENCLOSURE

SAFETY	FOR ACTION/INFORMATION	ENVIRONMENTAL
ASSIGNED AD:		ASSIGNED AD: V. MOORE (LTR)
BRANCH CHIEF: (6) Schwenger		BRANCH CHIEF:
PROJECT MANAGER: Neighbors		PROJECT MANAGER:
LICENSING ASSISTANT: Sheppard		LICENSING ASSISTANT:
		B. HARLESS

INTERNAL DISTRIBUTION			
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<input checked="" type="checkbox"/> OELD		<input checked="" type="checkbox"/> LAINAS	
GOSSICK & STAFF	ENGINEERING	<input checked="" type="checkbox"/> IPPOLITO	
HANAUER	KNIGHT	<input checked="" type="checkbox"/> F. ROSA	ENVIRO TECH. ERNST
MTPC	BOSNAK		BALLARD
CASE	SIHWELL	OPERATING REACTORS	YOUNGBLOOD
<input checked="" type="checkbox"/> BOYD	PAWLICKI	<input checked="" type="checkbox"/> STELLO	
		<input checked="" type="checkbox"/> EISENHUT	
PROJECT MANAGEMENT	REACTOR SAFETY	<input checked="" type="checkbox"/> SHAO	SITE TECH. GAMMILL (2)
SKOVHOLT	ROSS	<input checked="" type="checkbox"/> BAER	
P. COLLINS	<input checked="" type="checkbox"/> NOVAK	<input checked="" type="checkbox"/> BUTLER	
HOUSTON	<input checked="" type="checkbox"/> ROSZTGCZY	<input checked="" type="checkbox"/> GRIMES	
MELTZ	<input checked="" type="checkbox"/> CHECK		SITE ANALYSIS VOLLMER
HELTEMES			BUNCH
SK	AT&I		<input checked="" type="checkbox"/> J. COLLINS
	SALTZMAN		KREGER
	RUTBERG		

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DUKE POWER COMPANY

Regulatory Docket File

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

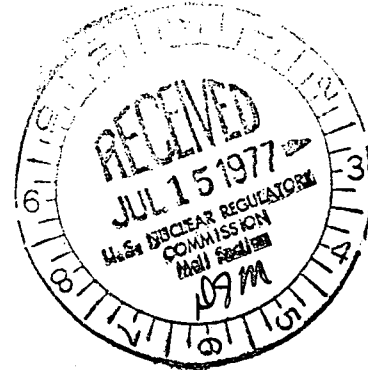
July 11, 1977

TELEPHONE: AREA 704
373-4083

Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. A. Schwencer, Chief
Operating Reactor Branch #1

Reference: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287



Dear Sir:

The B&W Topical Report BAW-10103, "ECCS Analysis of B&W's 177 FA Lowered-Loop NSS", contains the ECCS analysis applicable to Oconee Units 1, 2 and 3. Recently, a discrepancy has been identified in the value of the flow resistance factor associated with the reactor vessel inlet U-baffle assumed in the BAW-10103 analysis. The discrepancy relates to the fact that the BAW-10103-assumed values of these flow resistance factors were based on the original design characteristics of the U-baffle and did not take into account the modification of the U-baffle structure that was made since the original design. This was reported by letter dated June 10, 1977 which transmitted Reportable Occurrence Report RO-269/77-18.

A re-analysis of the ECCS performance has now been completed by B&W for the Oconee class reactors using the corrected flow resistance factors and utilizing the approved B&W ECCS Evaluation Model. This re-analysis is documented in a letter from Mr. James H. Taylor of B&W to Dr. R. L. Baer of the NRC, dated July 8, 1977. The analysis indicates that the ECCS evaluation results reported in BAW-10103 are conservative with respect to the re-analysis using the corrected flow resistance factors.

The Oconee Nuclear Station Technical Specifications include certain specified limits on core power distribution parameters which have been based on the linear heat rate limits established in BAW-10103. Since the ECCS evaluation results contained in BAW-10103 are found to be conservative compared to the results of the analysis using the corrected flow resistance factors, it is concluded that the current Technical Specifications limits for Oconee Unit 1, 2, and 3 and the proposed Technical Specification changes for Oconee 1 Cycle 4 and Oconee 2 Cycle 3 are conservatively valid.

Very truly yours,

William O. Parker, Jr.

PMA:ge

cc: Mr. N. C. Moseley

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