

AEC DISTRIBUTION FOR PART 50 DOCKET MATERIAL
(TEMPORARY FORM)

CONTROL NO: 8847

FILE: A/6

FROM: Duke Power Company Charlotte, N.C. 28201 Mr. A.C. Thies		DATE OF DOC 12-7-73	DATE REC'D 12-12-73	LTR X	MEMO	RPT	OTHER
TO: A. Giambusso		ORIG 1 signed	CC	OTHER	SENT AEC PDR	XXX	
					SENT LOCAL PDR	XXX	
CLASS XXX	UNCLASS	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-269		

DESCRIPTION:
Ltr reporting an abnormal occurrence at the Oconee Unit #1, trans the following.....

PLANT NAME: Oconee #1

ENCLOSURES:
Abnormal Occurrence Report No. UE-269/73-12
concerning failure of Engineered Safeguards Valve FDW-108 to close...

ACKNOWLEDGED
(1 cy encl rec'd)

DO NOT REMOVE

FOR ACTION/INFORMATION 12-13-73 JB

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| ✓ OGC, ROOM P-506A | SCHROEDER | GAMMILL | GEARIN (L) | SALTZMAN |
| ✓ MUNTZING/STAFF | ✓ MACCARY | KASTNER | ✓ GOULBOURNE (L) | B. HURT |
| CASE | ✓ KNIGHT | BALLARD | LEE (L) | PLANS |
| GIAMBUSSO | ✓ PAWLICKI | SPANGLER | MAIGRET (L) | MCDONALD |
| BOYD | ✓ SHAO | | SERVICE (L) | DUBE |
| MOORE (L) (BWR) | ✓ STELLO | ENVIRO | SHEPPARD (E) | INFO |
| ✓ DEYOUNG(L) (PWR) | ✓ HOUSTON | MULLER | SMITH (L) | C. MILES |
| SKOVHOLT (L) | ✓ NOVAK | DICKER | TEETS (L) | ✓ B. King |
| P. COLLINS | ✓ ROSS | KNIGHTON | WADE (E) | |
| | ✓ IPPOLITO | YOUNGBLOOD | WILLIAMS (E) | |
| ✓ REG OPR | ✓ TEDESCO | REGAN | WILSON (L) | |
| FILE & REGION(3) | ✓ LONG | PROJECT LDR | | |
| ✓ MORRIS | ✓ LAINAS | | | |
| ✓ STEELE | ✓ BENAROYA | HARLESS | | |
| | ✓ VOLLMER | | | |

EXTERNAL DISTRIBUTION

- | | | |
|---|------------------------------|------------------------|
| ✓ 1 - LOCAL PDR <u>Walhalla, S.C.</u> | (1)(2)(10)-NATIONAL LAB'S | 1-PDR-SAN/LA/NY |
| ✓ 1 - DTIE(ABERNATHY) | 1-ASLBP(E/W Bldg, Rm 529) | 1-GERALD LELLOUCHE |
| ✓ 1 - NSIC(BUCHANAN) | 1-W. PENNINGTON, Rm E-201 GT | BROOKHAVEN NAT. LAB |
| 1 - ASLB(YORE/SAYRE/
WOODARD/"H" ST. | 1-CONSULTANT'S | 1-AGMED(Ruth Gussman) |
| ✓ 16 - CYS ACRS HOLDINGS Sent to Goulbourne
12-13-73 | NEWMARK/BLUME/AGBABIAN | RM-B-127, GT. |
| | 1-GERALD ULRIKSON...ORNL | 1-RD..MULLER..F-309 GT |

DUKE POWER COMPANY

POWER BUILDING

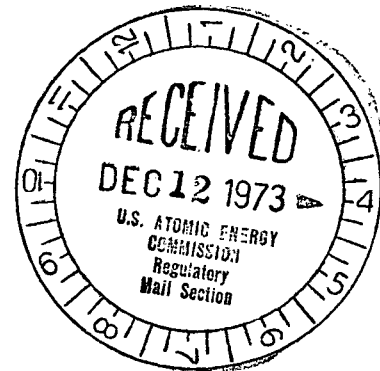
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

December 7, 1973

Mr. Angelo Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545



Re: Oconee Unit 1
Docket No. 50-269

Dear Mr. Giambusso:

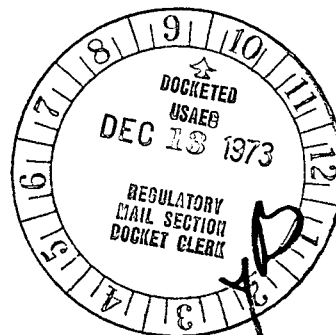
Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Unusual Event Report UE-269/73-12, "Engineered Safeguards Valve FDW-108."

Very truly yours,

[Handwritten signature]
A. C. Thies
for

ACT:vr
Attachment

cc: Mr. Norman C. Moseley



DUKE POWER COMPANY
OCONEE NUCLEAR STATION - UNIT 1
UNUSUAL EVENT REPORT UE-269/73-12
ENGINEERED SAFEGUARDS VALVE VDW-108

Description of the Incident

On November 8, 1973, Engineered Safeguards Valve FDW-108 failed to close during a monthly test of Engineered Safeguards System Channel 6. This valve serves to isolate the sample line from No. 1B steam generator in the event of Engineered Safeguard System actuation. Regulatory Operations, Region II, was verbally notified of the incident on November 8, 1973.

Corrective Action

Instrument technicians checked out the system immediately, but no problem was found. The valve operated properly on six successive trials. The valve was checked once a day for proper operation during November 9-11, 1973 and functioned properly each time. On November 12, 1973, local observation revealed some hesitation in valve operation. During successive operation, it was noted that the hesitation seemed to be the result of the slow operation of a solenoid which functions to admit opening air to the pneumatic operator of a valve. This solenoid was replaced and the valve FDW-108 was successfully tested for proper operation.

Safety Analysis

FDW-108 is a pneumatic isolation valve on the sample line from steam generator 1B and closes on signal from Channel 6 of the Engineered Safeguards System. Engineered Safeguards System Channel 6 is actuated from high reactor building pressure. On the same sample line, there is an electric motor-operated isolation valve, FDW-107, which is actuated by redundant Engineered Safeguards System Channel 5. In the event that FDW-108 had failed to close during Engineered Safeguards System actuation, reactor building isolation would have been provided by the redundant valve FDW-107 actuated by Engineered Safeguards Channel 5. This incident did not affect the safe operation of the plant or the health and safety of the public.