

AEC CONTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)

CONTROL NO: 8328

FILE: \_\_\_\_\_

<b>FROM:</b> Duke Power Company Charlotte, N.C. 28201 Mr. A.C. Thies			DATE OF DOC 11-12-73	DATE REC'D 11-19-73	LTR X	MEMO	RPT	OTHER
<b>TO:</b> A. Giambusso			ORIG 1 signed	CC	OTHER	SENT AEC PDR XXX SENT LOCAL PDR XXX		
CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:			
	XXX			1	50-269			

**DESCRIPTION:**  
Ltr. re their 10-16-73 ltr...adv that their ltr was incorrectly designated AO-269/73-5 and submitting a report which shows this correction and (2) additional corrections to the text of the report.....trans the following.

**ENCLOSURES:**  
Additional corrections to the text of Report AO-269/73-5.

**ACKNOWLEDGED**  
**DO NOT REMOVE**

PLANT NAME: Oconee

FOR ACTION/INFORMATION 11-19-73 JB

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✓ MUNTZING/STAFF	✓ MACCARY	KASTNER	✓ GOULBOURNE (L)	B. HURT
✓ CASE	✓ KNIGHT	BALLARD	LEE (L)	<u>PLANS</u>
GIAMBUSSO	✓ PAWLICKI	SPANGLER	MAIGRET (L)	MCDONALD
BOYD	✓ SHAO		SERVICE (L)	DUBE
MOORE (L)(BWR)	✓ STELLO	<u>ENVIRO</u>	SHEPPARD (E)	<u>INFO</u>
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FILE & REGION(3)	✓ LONG	PROJECT LDR		
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✓ 16 - CYS ACRS HOLDING	NEWMARK/BLUME/AGBABIAN	RM-B-127, GT.
11-19-73	1-GERALD ULRIKSON...ORNL	1-RD..MULLER..F-309 GT

A/O  
LJ

DUKE POWER COMPANY

POWER BUILDING

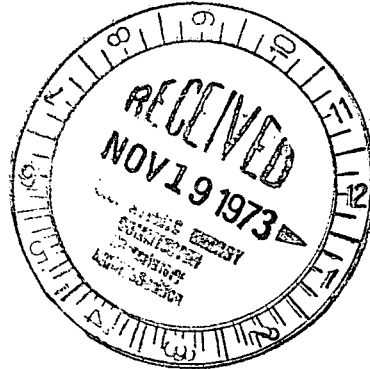
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES  
SENIOR VICE PRESIDENT  
PRODUCTION AND TRANSMISSION

P. O. Box 2178

November 12, 1973

Mr. Angelo Giambusso  
Deputy Director for Reactor Projects  
Directorate of Licensing  
Office of Regulation  
U. S. Atomic Energy Commission  
Washington, D. C. 20545



Re: Oconee Unit 1  
Docket No. 50-269

Dear Mr. Giambusso:

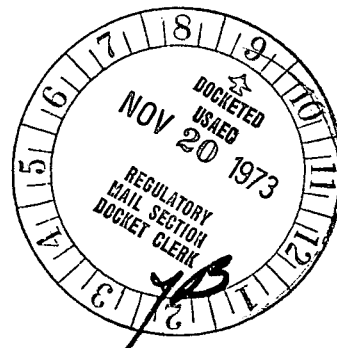
My letter of October 16, 1973 transmitted to you an Abnormal Occurrence Report entitled, "Control Rod Withdrawal Limits." This report was incorrectly designated Abnormal Occurrence Report No. AO-269/73-5. The correct designation is AO-269/73-6. Attached is a report which shows this correction and two additional corrections to the text of the report. These corrections are indicated by vertical lines in the margins.

Very truly yours,

A. C. Thies

ACT:vr  
Attachment

cc: Mr. Norman C. Moseley, Director  
Directorate of Regulatory Operations  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303



DUKE POWER COMPANY  
OCONEE NUCLEAR STATION - UNIT 1  
ABNORMAL OCCURRENCE REPORT AO-269/73-6  
CONTROL ROD WITHDRAWAL LIMITS

Introduction

As a limiting condition for operation, control rods must be maintained within the withdrawal limits specified in Figure 3.5.2-1 of the Oconee Nuclear Station Technical Specifications. On October 5, 1973, it was determined that Oconee Unit 1 control rods were operated such that these withdrawal limits were exceeded. Regulatory Operations, Region II, was verbally notified of the incident on October 5, 1973.

Description of the Incident

Shortly after midnight on October 5, 1973, Oconee Unit 1 was at 95 percent full power and control rods were moving into the core to compensate for transient xenon. At this time, it was determined that the control rod withdrawal limits contained in Technical Specification 3.5.2 had been exceeded. Control group 5 was fully withdrawn; control group 6 was 81 percent withdrawn and group 7 was 10 percent withdrawn. The rod index for the configuration was 191. The minimum rod index permitted at 95 percent power is 204.

Corrective Action

As soon as it was determined that rod withdrawal limits had been exceeded, power was reduced until the rod configuration was within the permissible operating range. This was completed in less than 15 minutes after it had been determined that the withdrawal limits had been exceeded.

The reactor was operated at reduced power for approximately 3.5 hours while the soluble poison concentration was adjusted. The reactor was then returned to 95 percent full power keeping the control rods within the permissible operating region.

Analyses are presently being performed to determine if operation outside

the rod withdrawal limits for a short period of time is permissible provided action is taken immediately to return the control rods within the limits specified by Technical Specification 3.5.2. If this is permissible, a request to change this technical specification will be submitted to the Atomic Energy Commission.

#### Safety Analysis

The control rod position limits defined in Specification 3.5.2.5c, in conjunction with the quadrant power tilt limits set forth in Specification 3.5.2.5, ensure that design peak heat rate criteria are not exceeded during normal operation when including the effects of potential fuel densification. Since quadrant tilt was well within the allowable limits, and since there was no sustained steady-state operation outside the withdrawal limits, it can be concluded that this incident had no adverse affect on the safe operation of the unit.