

A10/05/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-269

REC: OREILLY J P
NRC

ORG: PARKER W O
DUKE PWR

DOC DATE: 09/29/78
DATE RCVD: 10/04/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-269/78-020) DN 08/29/78 CONCERNING
VALVE LPSW-21 (RECU IE COOLING WATER DISCHARGE VAVLE) FAILED TO OPERATE
PROPERLY, DUE TO MISALIGNMENT OF GEARING BETWEEN THE VALVE AND ITS
OPERATOR...W/ATT.

PLANT NAME: OCONEE - UNIT 1

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: *al*

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF ORB#4 BC**W/4 ENCL

INTERNAL: REG FILE**W/ENCL
I & E**W/2 ENCL
I & C SYSTEMS BR**W/ENCL
NOVAK/CHECK**W/ENCL
AD FOR ENG**W/ENCL
HANAUER**W/ENCL
AD FOR SYS & PROJ**W/ENCL
ENGINEERING BR**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
EMERGENCY PLAN BR**W/ENCL
EEB**W/ENCL
PLANT SYSTEMS BR**W/ENCL
AD FOR PLANT SYSTEMS**W/ENCL
REACTOR SAFETY BR**W/ENCL
VOLLMER/BUNCH**W/ENCL
POWER SYS BR**W/ENCL

EXTERNAL: LPDR'S
WALHALLA, SC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 44 ENCL 44
SIZE: 1F+1P+1P

CONTROL NBR: 780730165

***** THE END *****

A104
clp

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

September 29, 1978

TELEPHONE: AREA 704
373-4083

US NRC
SERVICES
DISTRIBUTION
BRANCH

1978 OCT 4 PM 3 08

RECEIVED DISTRIBUTION
SERVICES UNIT

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

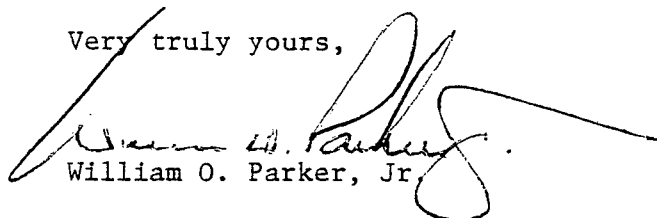
RE: Oconee Unit 1
Docket No. 50-269

REGULATORY DOCKET FILE COPY

Dear Mr. O'Reilly:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-269/78-20.

Very truly yours,



William O. Parker, Jr.

RLG:scs
Attachment

cc: Director, Office of Management Information
and Program Control

REGULATORY DOCKET FILE COPY

780730165

A002/S *
1/1

DUKE POWER COMPANY
OCONEE UNIT 1

Report Number: RO-269/78-20

Report Date: September 29, 1978

Occurrence Date: August 29, 1978

Facility: Oconee Unit 1, Seneca, South Carolina

Identification of Occurrence: Reactor Building Cooling Unit Discharge Valve Inoperable.

Condition Prior to Occurrence: 57% Full Power

Description of Occurrence:

On August 29, 1978, motor operated Engineered Safeguard (ES) valve LPSW-21 was determined to be inoperable in the closed position with the position indicator showing the valve in the intermediate position. Valve LPSW-21 is the cooling water discharge valve for the Reactor Building Cooling Unit (RBCU) 1B. The valve was inspected and the limit switch position indicator was adjusted to give proper position indication. However, the valve still did not operate so the motor operator was removed from service to allow mechanical disassembly. No problems were found that would cause the motor operator not to operate. The valve and operator were reassembled, installed, and verified to be operable.

Apparent Cause of Occurrence:

The limit switch was found to be out of adjustment and was corrected. The probable cause of the inoperable valve was the motor operator's clutch lever not fully engaging the gearing in the motor mode. The valve operated in the manual (handwheel) mode and the motor operator operated when removed from the valve.

Analysis of Occurrence:

There are three trains of Reactor Building cooling installed in Oconee Unit 1. Specification 3.3.4 requires all three operable if the reactor is critical. However, Specification 3.3.6 allows a seven day maintenance period on one RBCU under certain conditions. These conditions were met in that the reactor building spray systems were fully operable and available to mitigate the consequences of any postulated accident. Thus the health and safety of the public were assured.

Corrective Action:

The valve's limit switch was adjusted to indicate the valve's proper position. The motor operator was completely inspected to assure proper operation. After reassembly, the entire valve operator assembly was satisfactorily tested. No parts were replaced during the inspection of this valve.

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: [] [] [] [] [] [] [] [] [] [] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

[0] [1] [S] [C] [N] [E] [E] [1] [2] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [3] [4] [1] [1] [1] [1] [4] [] [] [5]
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CONT [0] [1] REPORT SOURCE [L] [6] [0] [5] [0] [0] [0] [2] [6] [9] [7] [0] [8] [2] [9] [7] [8] [8] [0] [9] [2] [9] [7] [8] [9]
7 8 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

[0] [2] During steady state operation, valve LPSW-21 (RBCU IB cooling water
[0] [3] discharge valve) failed to operate properly. The valve was closed but
[0] [4] indicated intermediate. The valve would not operate in the motor mode, but
[0] [5] would operate in the manual mode. Redundant trains of RB cooling were avail-
[0] [6] able, as well as building spray. No adverse effects to the public could
[0] [7] result.
[0] [8]

[0] [9] SYSTEM CODE [S] [B] (11) CAUSE CODE [E] (12) CAUSE SUBCODE [B] (13) COMPONENT CODE [Y] [A] [L] [V] [E] [X] (14) COMP. SUBCODE [F] (15) VALVE SUBCODE [D] (16)
7 8 9 10 11 12 13 14 15 16 17
[17] LER/RO REPORT NUMBER [7] [8] (21) [] (22) SEQUENTIAL REPORT NO. [0] [2] [0] (24) [] (25) OCCURRENCE CODE [0] [3] (28) [L] (30) [] (31) REVISION NO. [0] (32)
ACTION TAKEN [X] (18) FUTURE ACTION [Z] (19) EFFECT ON PLANT [Z] (20) SHUTDOWN METHOD [Z] (21) HOURS [0] [0] [0] [0] (22) ATTACHMENT SUBMITTED [Y] (23) NPRO-4 FORM SUB. [Y] (24) PRIME COMP. SUPPLIER [N] (25) COMPONENT MANUFACTURER [] (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

[1] [0] The apparent cause was a misalignment the gearing between the valve and its
[1] [1] operator. No problems were discovered during inspections of both components.
[1] [2] The valve and motor were reassembled and proper operation was verified.
[1] [3]
[1] [4]

[1] [5] FACILITY STATUS [E] (28) % POWER [0] [5] [7] (29) OTHER STATUS [NA] (30) METHOD OF DISCOVERY [A] (31) DISCOVERY DESCRIPTION [Operator Observation] (32)
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1] [6] ACTIVITY CONTENT [Z] (33) [Z] (34) [NA] (35) AMOUNT OF ACTIVITY [NA] (36) LOCATION OF RELEASE [NA] (37)
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1] [7] PERSONNEL EXPOSURES NUMBER [0] [0] [0] (37) TYPE [Z] (38) DESCRIPTION [NA] (39)
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1] [8] PERSONNEL INJURIES NUMBER [0] [0] [0] (40) DESCRIPTION [NA] (41)
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1] [9] LOSS OF OR DAMAGE TO FACILITY TYPE [Z] (42) DESCRIPTION [NA] (43)
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[2] [0] PUBLICITY ISSUED [N] (44) DESCRIPTION [NA] (45)
7 8 9 10 11 12 13 14 15 16 17 18 19 20