

50-269

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FILE NUMBER
INCIDENT REPORT

TO:
Mr. Norman G. Moseley

FROM:
Duke Power Company
Charlotte, North Carolina
Mr. William O. Parker, Jr.

DATE OF DOCUMENT
4/15/77
DATE RECEIVED
4/26/77

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DESCRIPTION

Ltr. trans the following:

PLANT NAME: (1-P)
Oconee Unit No. 1

RJL

ENCLOSURE

Licensee Event Report (Ro. 50-269/77-10) on 3/16/77 concerning feedwater containment. isolation valve 1FDW-108 being inoperable...

DO NOT REMOVE
ACKNOWLEDGED

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

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LPDR: <i>Walters, SC</i>			
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77170116

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.,
VICE PRESIDENT
STEAM PRODUCTION

April 15, 1977

TELEPHONE: AREA 704
373-4083

Mr. Norman C. Moseley, Director
U. S. Nuclear Regulatory Commission
Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

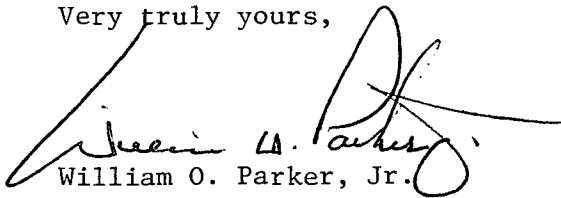
Re: Oconee Unit 1
Docket No. 50-269



Dear Mr. Moseley:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-269/77-10.

Very truly yours,


William O. Parker, Jr.

LJB:ge
Attachment

cc: Director, Office of Management Information
and Program Control

Regulatory

File Cyaj

771170116

DUKE POWER COMPANY
OCONEE UNIT 1

Report No.: RO-269/77-10

Report Date: April 15, 1977

Occurrence Date: March 16, 1977

Facility: Oconee Unit 1, Seneca, South Carolina

Identification of Occurrence: Feedwater containment isolation valve 1FDW-108 inoperable

Conditions Prior to Occurrence: Unit at 91 percent full power

Description of Occurrence:

On March 16, 1977, following chemical sampling of the "1B" once-through steam generator, feedwater containment isolation valve 1FDW-108 failed to close under system pressure. This valve, located outside the Reactor Building, is part of the chemical sampling system and provides containment isolation following an ES actuation. The redundant valve, 1FDW-107, located inside the Reactor Building was closed and locked as required by Oconee Technical Specification 3.6.4.b.2.

Apparent Cause of Occurrence:

Examination of the valve indicated that the inability of the valve to cycle properly resulted from the hardening of the valve packing.

Analysis of Occurrence:

Valve 1FDW-108 was properly isolated in compliance with Technical Specification 3.6.4.b.2 by securing redundant valve 1FDW-107 in the closed position. In the event that containment isolation had been required prior to securing valve 1FDW-107, valve 1FDW-107 would have closed upon an ES actuation. Containment integrity was not affected by this incident and it is thus concluded that the health and safety of the public were not affected.

Corrective Action:

Valve 3FDW-108 was repacked and its operability verified. To prevent recurrence of this incident, the following action will be taken.

A program will be developed to periodically adjust or replace packing on the FDW-106 and FDW-108 valves on all three units. This program will include examination and, as necessary, replacement of the packing on these valves during each unit's next refueling outage. A review will also be made of different types of packing which could eliminate hardening problems.

To determine if these valves need to be changed to a different design and/or different operator, the operation and maintenance histories of these valves are being reviewed and evaluated.

APR 17 10 15 AM '77

U.S.A.E.C.
REGULATORY OPERATIONS
REGION II
ATLANTA, GA.

APR 18 10 17 AM '77