

**NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL
(TEMPORARY FORM)**

CONTROL NO: 306

FILE: INCIDENT REPORT FILE

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| FROM: Duke Power Co. Charlotte, N.C. W.O. Parker, Jr. | | DATE OF DOC 1-6-76 | DATE REC'D 1-13-76 | LTR XXX | TWX | RPT | OTHER |
| TO: N.C. Moseley | | ORIG 1 Signed | CC 0 | OTHER | SENT AEC PDR SENT LOCAL PDR | | XXX XXX |
| CLASS | UNCLASS XXX | PROP INFO | INPUT | NO CYS REC'D 1 | DOCKET NO: 50-269 | | |

DESCRIPTION:
Letter trans the following.....

PLANT NAME: Oconee # 1

ENCLOSURES:
Unusual Event Report # 75-10, on 12-2-75,
Concerning Inoperable Reactor Building
Isolation Valve.....

(1 Copy Received)

FOR ACTION/INFORMATION

SAB 1-15-76

- | | | | |
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ACKNOWLEDGED**

INTERNAL DISTRIBUTION

- | | | | | |
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| <u>REG FILE</u> NRC PDR OGC, ROOM P-506A GOSSICK/STAFF CASE | <u>TECH REVIEW</u> SCHROEDER MACCARY KNIGHT PAWLICKI SHAO ** STELLO ** HOUSTON ** NOVAK ROSS IPPOLITO TEDESCO J. COLLINS LAINAS BENAROYA VOLLMER | DENTON GRIMES GAMMILL KASTNER BALLARD SPANGLER ENVIRO MULLER DICKER KNIGHTON YOUNGBLOOD REGAN PROJECT LDR HARLESS | <u>LIC ASST</u> R. DIGGS (L) H. GEARIN (L) E. GOULBOURNE (L) P. KREUTZER (E) J. LEE (L) M. RUSHBROOK (L) S. REED (E) M. SERVICE (L) S. SHEPPARD (L) M. SLATER (E) H. SMITH (L) S. TEETS (L) G. WILLIAMS (E) V. WILSON (L) R. INGRAM (L) M. DUNCAN (E) | <u>A/T IND.</u> BRAITMAN SALTZMAN MELTZ <u>PLANS</u> MCDONALD CHAPMAN DUBE (Ltr) E. COUPE PETERSON HARTFIELD (2) KLECKER EISENHUT WIGGINTON F. WILLIAMS HANAUER |
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EXTERNAL DISTRIBUTION

- | | | |
|--|--------------------------------|---|
| 1 - LOCAL PDR Walhalla, S.C. | 1 - NATIONAL LABS | 1 - PDR-SAN/LA/NY |
| 1 - TIC (ABERNATHY) (1)(2)(10) | 1 - W. PENNINGTON, Rm E-201 GT | 1 - BROOKHAVEN NAT LAB |
| 1 - NSIC (BUCHANAN) | 1 - CONSULTANTS | 1 - G. ULRIKSON, ORNL |
| 1 - ASLB | NEWMARK/BLUME/AGBABIAN | 1 - AGMED (RUTH GUSSMAN) Rm B-127 GT |
| 1 - Newton Anderson | | 1 - J. D. RUNKLES, Rm E-201 GT |
| 16 - ACRS SENT TO LIC ASST S. Sheppard | | |
| ** SEND ONLY TEN DAY REPORTS | | |

Handwritten initials/signature

DUKE POWER COMPANY

POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

Regulatory

File Copy

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

January 6, 1976

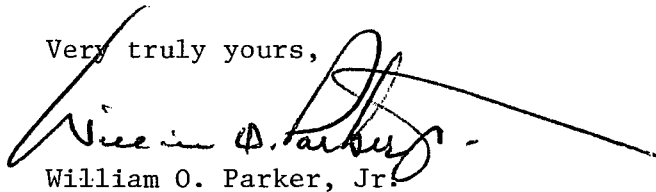
Mr. Norman C. Moseley, Director
U. S. Nuclear Regulatory Commission
Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Re: Oconee Unit 1
Docket No. 50-269

Dear Mr. Moseley:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station
Technical Specifications, please find attached Unusual Event
Report UE-269/75-10.

Very truly yours,

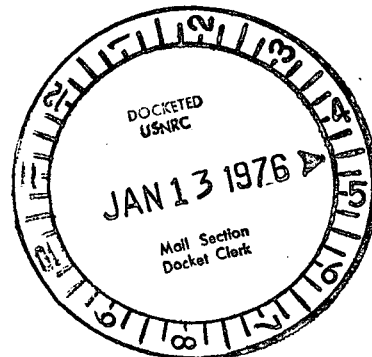
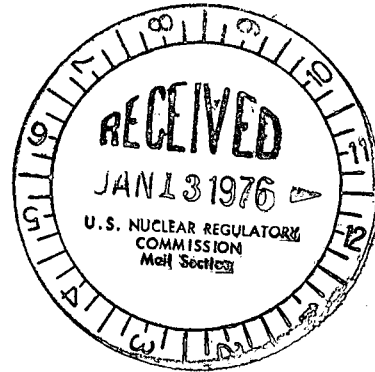


William O. Parker, Jr.

EDB:mmb

Attachment

CC Mr. Benard C. Rusche



DUKE POWER COMPANY

OCONEE UNIT 1

Report No.: UE-269/75-10

Report Date: January 6, 1976

Event Date: December 2, 1975

Facility: Oconee Unit 1, Seneca, South Carolina

Identification of Event: Inoperable Reactor Building isolation valve

Conditions Prior to Event: Unit at 75% full power

Description of Event:

On December 2, 1975, during the performance of the Reactor Building Isolation Channel 6 on-line test, IP/0/A/0310/13C, the Engineered Safeguards(ES) pressurizer sample isolation valve IRC-7 failed to close as required. After several attempts, however, the valve was successfully closed within approximately 90 seconds. Proper operation of this valve had been previously verified on November 4, 1975.

Apparent Cause of Event:

Valve IRC-7 is one of several ES actuated valves that had recently been painted. It is a piston-operated valve and apparently some paint had gotten on the piston stem and prevented the valve from functioning. After the paint was removed and the stem lubricated, the valve operated normally after several cycles.

Analysis of Event:

Valves IRC-5 and IRC-6 are parallel valves located in the two pressurizer sample lines inside containment and are redundant to valve IRC-7 located outside containment downstream from where the two sample lines join. These two redundant valves were operable and would have isolated the sample line had an ES actuation occurred. It is concluded that the health and safety of the public was not affected by this incident.

Corrective Action:

All ES valves which have recently been painted will be inspected for the presence of paint on the operating piston and appropriate corrective action will be taken if necessary. This inspection will be completed by January 15, 1976.