C 06/05/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-269/270/287

REC: CASE E G
NRC

ORG: PARKER W O
DUKE PWR

DOCDATE: 06/02/78
DATE RCVD: 06/05/78

DOCTYPE: LETTER NOTARIZED: YES

COPIES RECEIVED
LTR 3 ENCL 40

SUBJECT:
FORWARDING LIC NOS DPR-38, 47 & 55 APPL FOR AMEND: APPENDIX A TECH SPEC PROPOSED CHANGE CONCERNING CHANGE TO THE INSPEC SCHEDULE FOR REACTOR COOLANT OUTLET NOZZLES DUE TO AN ERROR THAT OCCURRED DURING LAST REFUELING OUTAGE... NOTARIZED 06/02/78... W/ATT LIC FEES

PLANT NAME: OCONEE - UNIT 1
OCONEE - UNIT 2
OCONEE - UNIT 3

REVIEWER INITIAL: XJM
DISTRIBUTER INITIAL:

***************** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****************

NOTES:
1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LICENSE.
(DISTRIBUTION CODE A001)

FOR ACTION: BR CHIEF ORB #4 CHIEF**W/7 ENCL

INTERNAL:
REG FILE**W/ENCL
I & E**W/2 ENCL
HANAUER**W/ENCL
AD FOR OPER TECH**W/ENCL
REACTOR SAFETY BR**W/ENCL
EEB**W/ENCL
J. MCGOUGH**W/ENCL
NRC PDR**W/ENCL
OELD**LTR ONLY
CORE PERFORMANCE BR**W/ENCL
ENGINEERING BR**W/ENCL
PLANT SYSTEMS BR**W/ENCL
EFFLUENT TREAT SYS**W/ENCL

EXTERNAL:
LPDR'S
WALHALLA, SC**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

$$
$ CHECK NBR: 196,067 $
$ AMOUNT: $4,800.00 $
$ CHECK AND COPY OF TRANSMITTAL LTR ADVANCED $
$ TO W. MILLER (LFMB) (06/05/78) UPON RECIEPT $
$$

A/2

DISTRIBUTION: LTR 40 ENCL 39
SIZE: 2P+1P

CONTROL NBR: 781560092

************************************* THE END *************************************

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

June 2, 1978

Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. R. Reid, Chief
Operating Reactors Branch #4

Reference: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sirs:

Pursuant to 10CFR50, §50.90, please find attached a proposed license amendment which incorporates a change to Oconee Nuclear Station Technical Specifications.

Specification 4.2.2 concerns the inspection schedule of portions of the reactor coolant system. A change to the inspection schedule for reactor coolant outlet nozzles is requested due to an error that occurred during the last Oconee Unit 3 refueling outage. During this outage, one RC outlet nozzle was inspected but an incorrect procedure was utilized. The procedure used a UT beam angle of approximately 75 degrees instead of an angle of essentially 90 degrees as required by I-2320 of Section XI of the ASME Code. This error resulted in 10 to 20 percent of the weld volume not being examined. Due to scheduling conflicts is not practical to perform this inspection again during the forthcoming Oconee Unit 3 refueling outage and stay within the approximately 40 month inspection interval required. Therefore, it is requested that the change be made to allow this nozzle to be re-inspected at a future refueling outage. The forthcoming refueling outage is for Cycle 4, therefore, the subsequent refueling outage would be for Cycle 5 and is so stated in the proposed revision to the specification.

This proposed license amendment is considered to constitute one Class III license fee request and two Class I license fee requests since they involve a single issue pertaining to three identical units. Accordingly, the enclosed check in the amount of $4,800 is remitted for the license amendment fee.

Very truly yours,

William O. Parker, Jr.

RLG:scs
Attachment

A001
S 1/40

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47 and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 2nd day of June, 1978.

Vivian B Rakkins
Notary Public

My Commission Expires:

Feb. 15, 1982

4.2 REACTOR COOLANT SYSTEM SURVEILLANCE

Applicability

Applies to the surveillance of the Reactor Coolant System pressure boundary.

Objective

To assure the continued integrity of the Reactor Coolant System pressure boundary.

Specification

4.2.1 Prior to initial unit operation, an ultrasonic test survey shall be made of Reactor Coolant System pressure boundary welds as required to establish preoperational integrity and baseline data for future inspections.

4.2.2 Post-operational inspections of components shall be made in accordance with the methods and intervals indicated in IS-242 and IS-261 of Section XI of the ASME Boiler and Pressure Vessel Code, 1970, including 1970 winter addenda, except as follows:

IS-261 Item	Component	Exception
1.4	Primary Nozzle to Vessel Welds	1 RC outlet nozzle to be inspected after approximately 3 1/3 years operation. 2nd RC outlet nozzle to be inspected after approx. 6 2/3 yrs. operation. 4 RC inlet nozzles and 2 core flooding nozzles to be inspected at or near end of interval*
3.3	Primary Nozzle to Safe End Welds	Not Applicable
4.3	Valve Pressure Retaining Bolting Larger than 2"	Not Applicable
6.1	Valve Body Welds	Not Applicable
6.3	Valve to Safe End Welds	Not Applicable
6.6	Integrally Welded Valve Supports	Not Applicable
6.7	Valve Supports & Hangers	Not Applicable

*For Oconee Unit 3 only, the 1st RC outlet nozzle will be inspected during Cycle 5 refueling outage.