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U.S. NUCLEAR REGULATORY COMMISSION

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50-269/270/287

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TO: Mr. Edson G. Case

FROM: Duke Power Co.
Charlotte, N. C. 28242
William O. Parker

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ENCLOSURE Request for revision to the Oconee Nuclear Station Tech Specs to provide a requirement for prompt written notification of certain reportable occurrences by telephone, mailgram or facsimile transmission...

FO ENCL.

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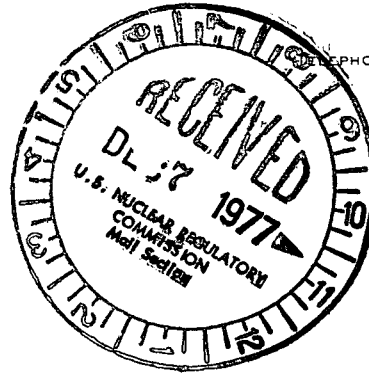
DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N.C. 28242

REGULATORY DOCKET FILE COPY

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

December 2, 1977

TELEPHONE: AREA 704
373-4083



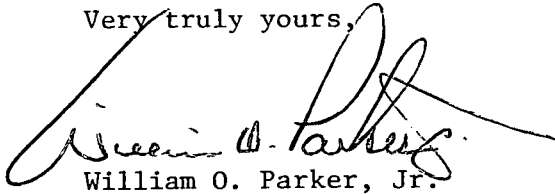
Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

RE: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

In response to your letter of November 10, 1977 and pursuant to 10CFR 50.90, a revision to the Oconee Nuclear Station Technical Specifications is requested to provide a requirement for prompt written notification of certain reportable occurrences by telephone, mailgram or facsimile transmission.

Very truly yours,


William O. Parker, Jr.

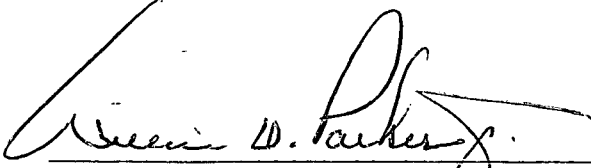
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December 2, 1977

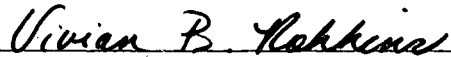
Page 2

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.



William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 2nd day of December, 1977.



Vivian B. Robbins
Notary Public

My Commission Expires:

Feb. 15, 1982

6.6.2 Non-Routine Reports

6.6.2.1 Reportable Occurrences

a. Prompt Notification with Written Followup

The types of events listed below shall be reported within 24 hours of discovery by telephone, and confirmed by telegraph, mailgram or facsimile no later than the first normal working day following the event to the Director, Office of Inspection and Enforcement, Region II, or his designate with a written followup report within two weeks to the Director, Office of Inspection and Enforcement, Region II (copy to the Director, Office of Management Information and Program Control).

- (1) Failure of the Reactor Protective System to trip, as required, when a monitored parameter reaches the setpoint specified as the limiting safety system setting in the Technical Specifications.
- (2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications.
- (3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary or primary containment.
- (4) Reactivity anomalies involving disagreement with predicted value of reactivity balance under steady-state conditions greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds, or if subcritical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or any unplanned criticality.
- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the Safety Analysis Report.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the Safety Analysis Report.
- (7) Conditions arising from natural or man-made events that, as a direct result of the event, require unit shutdown, operation of safety systems, or other protective measures required by Technical Specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the Safety Analysis Report or in the bases for the Technical Specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.