

FILE:

FROM: Duke Power Company Charlotte, N. C. 28201 A. C. Thies		DATE OF DOC 4-29-74	DATE REC'D 5-7-74	LTR X	MEMO	RPT	OTHER
TO: P. F. Collins		ORIG 1 signed	CC	OTHER	SENT AEC PDR X SENT LOCAL PDR X		
CLASS	UNCLASS XXXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: <u>50-269/270/287</u>		
DESCRIPTION: Ltr re our 4-5-74 ltr, trans the following:				ENCLOSURES: Addl. Info to the Requalification Program			
ACKNOWLEDGED				DO NOT REMOVE			
PLANT NAME: Oconee Units 1, 2, & 3				(1 cy rec'd)			

FOR ACTION/INFORMATION 5-7-74 GC

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Regulatory Docket File

DUKE POWER COMPANY

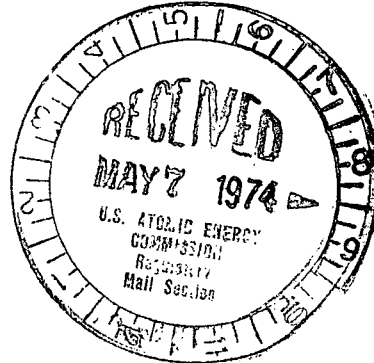
POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

April 29, 1974



Mr. Paul F. Collins, Chief
Operator Licensing Branch
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Collins:

In reply to your letter of April 5, 1974 we herewith submit additional information concerning the Oconee Nuclear Station Requalification Program for AEC Licensed Personnel.

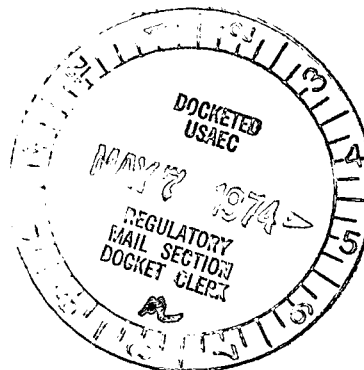
Very truly yours,

A.C. Thies

A. C. Thies

E.D.P.

ACT:th
Enclosure



DUKE POWER COMPANY
OCONEE NUCLEAR STATION

ADDITIONAL INFORMATION
REQUALIFICATION PROGRAM FOR
AEC LICENSED PERSONNEL IN
RESPONSE TO MR. P. F. COLLINS'
LETTER OF APRIL 5, 1974

- 2.1a This paragraph will be reworded as follows: "Critical approach from subcritical on source range instrumentation to critical at the point of adding heat on the intermediate range instrumentation."

Subsection 2.1 will be changed to read as follows: "A licensed Reactor Operator that performs, or a licensed Senior Reactor Operator that performs or directs, ten (10) reactivity changes from those listed below, in accordance with Section 4.0, On-The-Job Training, Subsection 4.2, meets the requirements of 10CFR55, Appendix A, Section 3(a).

- 3.0 Lectures are intended to be given on an annual basis over a period of approximately 14 weeks prior to the annual requalification exam. The lecture schedule following the requalification exam will be determined by the results of the exam. (i.e. categories in which individuals received less than 80%) The lecture schedule prior to each annual requalification examination will be determined based on weak areas shown from the preceding annual exam.

"Quality Assurance for Operation" is now a topic covered as part of the pre-license training and as such would be considered as a topic for requalification lectures.

Contingency meetings are an integral part of Nuclear plant operation and are normally scheduled as the need arises by plant management.

Periodic written or oral quizzes will be administered throughout the lecture program with a required passing grade of 80%. Lecture attendance will be based on 80% categorical grades from the Annual Requalification exam.

Any operator who scores less than 70% overall on the Annual Requalification exam shall be removed from licensed duties and placed in an accelerated requalification program. The individual will be reevaluated by a comprehensive written exam paralleling the exam in format and by an oral interview with the Training Supervisor to determine his knowledge level. The results of the oral and written exam will be forwarded to the Operating Engineer for determination of return to licensed duties.

Lecture attendance for all SRO and RO licensed personnel will be based on the results of the Annual Requalification exam. The second paragraph of Section 3.0, which addresses exemptions from attending certain lectures, will be deleted.

- 4.1 Documentation of the specified reviews is being provided. This documentation is filed in the individuals training records as per Section 6.0 Records, Subsection 6.1, Item (e) and (f).

Passing grade for written quizzes on reviews is 80%.

- 4.3 Evaluations of personnel will be made by several sources as per Section 5.0 Evaluations. The ultimate evaluation will be based on the operators performance of his licensed duties at the Oconee Nuclear Station.

The B&W simulator is considered the only existing simulator appropriate for use by Oconee personnel in meeting the requirements of Appendix A to 10 CFR Part 55. However, we retain the option to use other simulators that become available in the future and are approved by the AEC for use by Oconee personnel.

A further revision to the Oconee Requalification Program is hereby proposed to change the frequency of evaluations specified in Section 5.0 Evaluations, Subsections 5.2, and 5.4 to semi-annually vice quarterly. Evaluations of a semi-annual nature would be more suited to plant operations as well as meeting the intent of 10 CFR 55 Appendix A.