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FILE:

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Duke Power Co. Charlotte, N. C. William O. Parker, Jr.	9-3-75	9-8-75	XXX			

TO:	ORIG	CC	OTHER	SENT NRC PDR XXX SENT LOCAL PDR XX
Roger S. Boyd	1 Signed			

CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:
	XXX			1	50-269/270/287

DESCRIPTION:
Ltr. re our ltr dtd 7-18-74 & their ltr of 8-30-74 & withdrawal of proposed tech spec. withdrawn dn 2-27-75,& submittal in July 1975...
Advising they feel that proposal of a tech Spec for steam generator surveillance should be delayed until Section 11 is revised.....

ENCLOSURES:

ACKNOWLEDGED
DO NOT REMOVE

PLANT NAME: Oconee 1-2-3-

FOR ACTION/INFORMATION VCR 9-11-75

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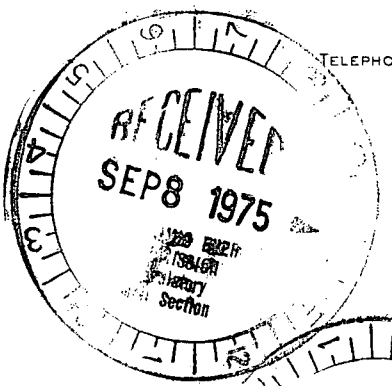
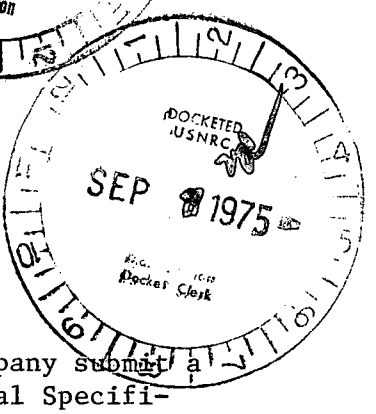
Regulatory Docket File

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083





September 3, 1975

Mr. Roger S. Boyd, Acting Director
Division of Reactor Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269, 50-270 and 50-287

Dear Mr. Boyd:

On July 18, 1974, the AEC requested that Duke Power Company submit a proposed revision to the Oconee Nuclear Station Technical Specifications to incorporate therein a program for steam generator tube surveillance consistent with Regulatory Guide 1.83, Revision 0. On August 30, 1974, a submittal in response to this request was made. Subsequently, on February 27, 1975 the proposed technical specification was withdrawn from further consideration and review. At that time, however, it was stated that a proposed technical specification would be resubmitted following re-evaluation of the matter and evaluation of the results of the Oconee 1 steam generator inspection.

The inspection of the Oconee 1 steam generators following the first cycle of operation, and evaluation of the results thereof, have been completed. Additionally, the Commission issued in July, 1975 Revision 1 to Regulatory Guide 1.83. It is understood, however, that criteria for steam generator surveillance are being developed for inclusion in Section XI of the ASME Boiler and Pressure Vessel Code and that it is intended that these criteria be issued in the Winter 1975 addenda to Section XI.

Based on all information currently available, as summarized above, it is felt that proposal of a technical specification for steam generator surveillance should be delayed until Section XI is revised to address this matter. In the interim, however, inspections on the Oconee units will be conducted in accordance with the provisions of Regulatory Guide 1.83.

Very truly yours,

William O. Parker, Jr.

DCH:vr

9528

to determine seismic loads may be used. The simplified procedure consists of consideration of the system as a single degree of freedom system and picking up a seismic response value from applicable floor response spectra, once the fundamental frequency of the system is determined. The floor response spectra should be obtained analytically (Section V.b) from the application of Regulatory Guide 1.60 design response spectra normalized to OBE level maximum ground acceleration at the foundation of the building housing the gaseous radwaste system.

(2) The allowable stresses to be used for the system support elements should be those given in the AISC Manual of Steel Construction, 7th edition 1970, including the one-third allowable stress increase provision for load combinations involving earthquake loads. For design of concrete foundations of the system, where applicable, use of the ACI 318-71 code with one-third increase in allowable stress for seismic loads is acceptable.

(3) The construction and inspection requirements for the support elements should comply with those stipulated in AISC or ACI Codes as appropriate.

b. Seismic Design Requirements for Buildings Housing Radwaste Systems

(1) Define input motion at the foundation of the building housing the radwaste systems. The motion should be defined by normalizing the Regulatory Guide 1.60 spectra to the OBE maximum ground acceleration selected for the plant.

A simplified analysis should be performed to determine appropriate seismic loads and floor response spectra pertinent to the location of the systems; i.e., an analysis of the building by a "several degrees of

freedom" mathematical model and the use of an approximate method to generate the floor response spectra for radwaste systems and the seismic loads for the buildings. No time history or dynamic analysis is required.

(2) The simplified method for determination of seismic loads for the building consists of (a) calculation of first several modal frequencies and participation factors for the building, (b) determination of modal seismic loads by item (1) input spectra, and (c) combination of modal seismic loads by the square root of the sum of squares (SRSS) rule.

(3) With regard to generation of floor response spectra for radwaste systems, methods such as the Biggs or other equivalent procedures which give approximate floor response spectra without need for performing a time history analysis may be used.

(4) The load factors and load combinations to be used for the building should be those given in the ACI-318-71 Code. The allowable stresses for steel components should be those given in the AISC Manual of Steel Construction, 7th edition, 1970.

(5) The construction and inspection requirements for the building elements should comply with those stipulated in the AISC or ACI Code as appropriate.

(6) The foundation media of structures housing the radwaste systems should not liquify during the Operating Basis Earthquake.

c. In lieu of the requirements and procedures defined above, optional shield structures constructed around and supporting the radwaste systems may be erected to protect the radwaste systems from effects of housing structural failure. If this option is adopted, the procedures described in Section V.b only need to be applied to the

shield structures while treating the rest of the housing structures as non-seismic Category I.

VI. Quality Assurance for Radioactive Waste Management Systems

A quality assurance program should be established that is sufficient to assure that the design, construction, and testing requirements are met. The quality assurance program should include the following:

a. Design and Procurement Document Control - Measures should be established to insure that the requirements of this position paper are specified and included in design and procurement documents and that deviations therefrom are controlled.

b. Control of Purchased Material, Equipment and Services - Measures should be established to assure that purchased material, equipment and construction services conform to the procurement documents.

c. Inspection - A program for inspection of activities affecting quality should be established and executed by, or for, the organization performing the activity to verify conformance with the documented instructions, procedures, and drawings for accomplishing the activity.

d. Handling, Storage, and Shipping - Measures should be established to control the handling, storage, shipping, cleaning and preservation of material and equipment in accordance with work and inspection instructions to prevent damage or deterioration.

e. Inspection, Test and Operating Status - Measures should be established to provide for the identification of items which have satisfactorily passed required inspections and tests.

f. Corrective Action - Measures should be established to assure that conditions adverse to quality, such as failures, malfunctions, deficiencies, deviations, defective material and equipment and nonconformances are promptly identified and corrected.

TABLE 1

EQUIPMENT CODES

EQUIPMENT	CODES			
	Design and Fabrication	Materials (2)	Welder Qualification And Procedure	Inspection And Testing
Pressure Vessels	ASME Code Section VIII, Div. 1	ASME Code Section II	ASME Code Section IX	ASME Code Section VIII, Div. 1
Atmospheric or 0-15 psig tanks	ASME Code (3) Section III, Class 3, or API 620 & 650, AWWA D-100	ASME Code (4) Section II	ASME Code Section IX	ASME Code (3) Section III, Class 3 or API 620;650 AWWA D-100
Heat Exchanger	ASME Code Section VIII, Div. 1 and TEMA	ASME Code Section III	ASME Code Section IX	ASME Code Section VIII, Div. 1
Piping and Valves	ANSI 31.1	ASTM or ASME Code Section II	ASME Code Section IX	ANSI B 31.1
Pumps	Manufacturers (1) Standards	ASME Code Section II or Manufacturers Standard	ASME Code Section IX (as required)	ASME (3) Section III Class 3; or Hydraulic Institute

Notes:

(1) Manufacturer's standard for the intended service. Hydrotesting should be 1.5 times the design pressure.

(2) Material Manufacturer's certified test reports should be obtained whenever possible.

(3) ASME Code stamp and material traceability not required.

(4) Fiberglass reinforced plastic tanks may be used in accordance with Part M, Section 10, ASME Boiler and Pressure Vessel Code, for applications at ambient temperature.

1047-105

PUBLIC VOUCHER FOR REFUNDS

Voucher No. File 50-269

Schedule No. 120

U. S. Nuclear Regulatory Commission, Division of Accounting
(Department or Establishment, Bureau or Office)

Location: Washington, D. C. 20555

Appropriation or Fund: 95X0200

PAID BY
10/20/75

To / Address:
Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Power Building
422 South Church Street
Charlotte, North Carolina 28242

Deposit received from the above-named depositor on _____, 19__

for _____

has been applied as herein stated and the balance indicated is returned herewith:

Amount of deposit	$
Applied as explained in "Remarks" below	
Balance authorized to be refunded	$ 166,920.00

Remarks:

Refund of Annual License Fee

Oconee 1

AA

3

(Sign original only) _____

Title _____

Refund by
- Check No. _____
- Cash, $_____ on _____ (Signature of payee) _____ (Sign original only)
- Other method, $_____, _____ (Describe)

U.S. GOVERNMENT PRINTING OFFICE

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

October 7, 1975

Mr. James D. Lincoln, Director
Division of Accounting
Office of the Controller
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Oconee Nuclear Station
Unit 1, Docket No. 50-269

Dear Mr. Lincoln:

In response to your notice of April 11, 1975, giving consideration for refunds of annual operating license fees, Duke Power Company submits herewith the following request for refund of fees paid for Oconee Nuclear Station, Unit 1:

License No.	DPR-38
Invoice No.	558-74
Inclusive Dates	2/6/74 - 2/5/75
Amount	$166,920 ✓
Previous Refunds	0

Total Refund Requested: $166,920

Very truly yours,

William O. Parker, Jr.

EDB:vr