

FROM: Duke Power Company Charlotte, N.C. 28201 Mr. A.C. Thies		DATE OF DOC 9-4-74	DATE REC'D 9-6-74	LTR X	TWX	RPT	OTHER
TO: A. Giambusso		ORIG 3 signed	CC	OTHER	SENT AEC PDR XXX SENT LOCAL PDR XXX		
CLASS	UNCLASS XXX	PROP INFO	INPUT XXX	NO CYS REC'D 40	DOCKET NO: <u>50-269</u> /270/287		

DESCRIPTION:
Ltr requesting a change to tech specs.....
concerning.....maintenance intervals on hatches
and isolation valves notarized 9-4-74

ENCLOSURES:

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: **Oconee**

FOR ACTION/INFORMATION

9-6-74 JB

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9-6-74		GT

BN

Regulatory Docket

DUKE POWER COMPANY

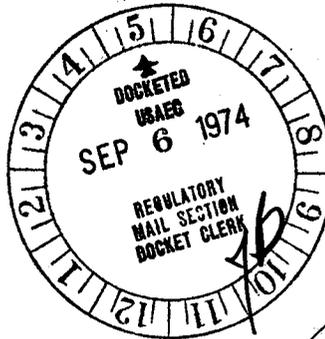
POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

September 4, 1974



Mr. Angelo Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287



Dear Mr. Giambusso:

Oconee Nuclear Station Technical Specification 3.6 requires containment integrity be maintained when all three of the following conditions exist:

1. Reactor coolant pressure is 300 psig or greater
2. Reactor coolant temperature is 200°F or greater
3. Nuclear fuel is in the core

Containment integrity, as defined in Section 1.7 of the Technical Specifications, exists when the following conditions are satisfied:

1. The equipment hatch is closed and sealed and both doors of the personnel hatch and emergency hatch are closed and sealed except as in 2 below.
2. At least one door of the personnel hatch and the emergency hatch is closed and sealed during refueling or during personnel passage through these hatches.
3. All non-automatic containment isolation valves and blind flanges are closed as required.
4. All automatic containment isolation valves are operable or locked closed.
5. The containment leakage determined at the last testing interval satisfies Specification 4.4.1.

9168

Mr. Angelo Giambusso
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There is no provision under the Technical Specifications which will permit preventative or corrective maintenance to be performed on the personnel or emergency hatches, or containment isolation valves during reactor operation. Also, there is no prescribed action should these components become inoperable during reactor operation.

Pursuant to 10CFR50.59, it is requested that Technical Specification 3.6 be revised to allow maintenance intervals on hatches and isolation valves for short periods of time. The proposed changes are shown on the attached replacement pages for the Oconee Technical Specifications.

Operation with a personnel or emergency hatch inoperable does not impair containment integrity since either door meets the design specifications for structural integrity and leak rate. The time limits imposed permit completion of maintenance action and the performance of a local leak rate test when required or the orderly shutdown and cooldown of the reactor. Timely corrective action for an inoperable containment isolation valve is also specified.

This request for a revision to the Oconee Technical Specifications encompasses the change requested in my letter dated July 26, 1974. Therefore, the request for change specified in that letter is hereby rescinded.

Very truly yours,



A. C. Thies

ACT:vr

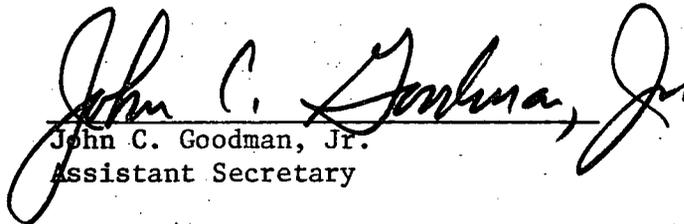
Mr. Angelo Giambusso
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A. C. THIES, being duly sworn, states that he is Senior Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Atomic Energy Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.



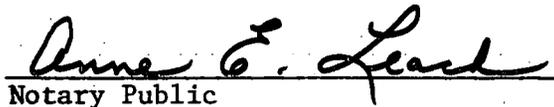
A. C. Thies
A. C. Thies, Senior Vice President

ATTEST:



John C. Goodman, Jr.
John C. Goodman, Jr.
Assistant Secretary

Subscribed and sworn to before me this 4th day of September, 1974.



Anne E. Leach
Notary Public

My Commission Expires:

September 16, 1974

3.6 REACTOR BUILDING

Applicability

Applies to the containment when the reactor is subcritical by less than 1% $\Delta k/k$.

Objective

To assure containment integrity during startup and operation.

Specification

- 3.6.1 Containment integrity shall be maintained whenever all three (3) of the following conditions exist:
- a. Reactor coolant pressure is 300 psig or greater
 - b. Reactor coolant temperature is 200°F or greater
 - c. Nuclear fuel is in the core
- 3.6.2 Containment integrity shall be maintained when the reactor coolant system is open to the containment atmosphere and the requirements for a refueling shutdown are not met.
- 3.6.3 The containment integrity shall be intact whenever positive reactivity insertions which would result in the reactor being subcritical by less than 1% $\Delta k/k$ are made by control rod motion or boron dilution.
- 3.6.4 Exceptions to 3.6.1, 3.6.2, and 3.6.3 shall be as follows:
- a. If either the personnel or emergency hatches become inoperable, except as a result of an inoperable door gasket, the hatch shall be restored to an operable status within 24 hours, or the reactor shall be in cold shutdown within the next 36 hours.

If a hatch is inoperable due to an inoperable door gasket:
 1. The remaining door of the affected hatch shall be closed and sealed.
 2. The hatch shall be restored to operable status within seven days or the reactor shall be in cold shutdown within the next 36 hours.
 - b. A containment isolation valve may be inoperable provided either:
 1. The inoperable valve is restored to operable status within four hours.
 2. The affected penetration is isolated within four hours by the use of a deactivated automatic valve secured and locked in the isolated position.

3. The affected penetration is isolated within four hours by the use of a closed manual valve or blind flange.
 4. The reactor is in the hot shutdown condition within 12 hours and cold shutdown within 24 hours.
- 3.6.5 The reactor building internal pressure shall not exceed 1.5 psig or five inches of Hg if the reactor is critical.
- 3.6.6 Prior to criticality following refueling shutdown, a check shall be made to confirm that all manual containment isolation valves which should be closed are closed and tagged.

Bases

The Reactor Coolant System conditions of cold shutdown assure that no steam will be formed and hence no pressure buildup in the containment if the Reactor Coolant System ruptures.

The selected shutdown conditions are based on the type of activities that are being carried out and will preclude criticality in any occurrence.

The reactor building is designed for an internal pressure of 59 psig and an external pressure 3.0 psi greater than the internal pressure. The design external pressure of 3.0 psi corresponds to a margin of 0.5 psi above the differential pressure that could be developed if the building is sealed with an internal temperature of 120°F with a barometric pressure of 29.0 inches of Hg and the building is subsequently cooled to an internal temperature of 80°F with a concurrent rise in barometric pressure to 31.0 inches of Hg. The weather conditions assumed here are conservative since an evaluation of National Weather Service records for this area indicates that from 1918 to 1970 the lowest barometric pressure recorded is 29.05 inches of Hg and the highest is 30.85 inches of Hg.

Operation with a personnel or emergency hatch inoperable does not impair containment integrity since either door meets the design specifications for structural integrity and leak rate. The time limits imposed permit completion of maintenance action and the performance of a local leak rate test when required or the orderly shutdown and cooldown of the reactor. Timely corrective action for an inoperable containment isolation valve is also specified.

When containment integrity is established, the limits of 10CFR100 will not be exceeded should the maximum hypothetical accident occur.

REFERENCES

FSAR, Section 5