

FILE: _____

FROM: Duke Power Comapny Charlotte, N.C. 28201 Mr. A.C. Thies		DATE OF DOC 8-30-74	DATE REC'D 9-5-74	LTR X	TWX	RPT	OTHER
TO: A. Giambusso		ORIG 3 signed	CC	OTHER	SENT AEC PDR SENT LOCAL PDR		XXX XXX
CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:		
	XXX		XXX	40	50-269/270/287		

DESCRIPTION:
Ltr trans the following....proposed Amdt...
notarized 8-30-74....

ENCLOSURES:
Amdt to the OL consist of Changes to Tech
Specs....concerning....Steam Generator
Tubing Surveillance.....

PLANT NAME: Oconee

ACKNOWLEDGED
DO NOT REMOVE

FOR ACTION/INFORMATION 9-6-74 JB

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DUKE POWER COMPANY

POWER BUILDING

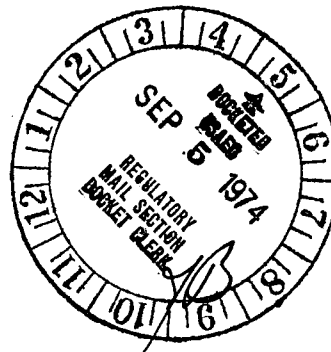
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

August 30, 1974

Mr. Angelo Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545



Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Giambusso:

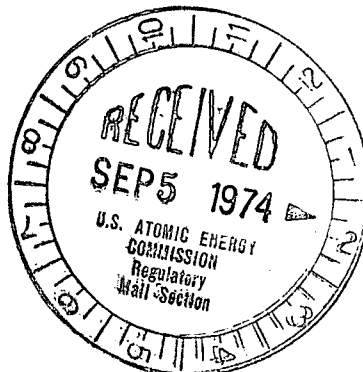
Pursuant to 10CFR50.90, please find attached a proposed amendment to the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, -47, and -55. This proposed amendment is submitted in response to Mr. Karl R. Goller's letter of July 18, 1974.

This proposed amendment incorporates into the Oconee Technical Specifications a program for the surveillance of steam generator tubing.

Very truly yours,

A. C. Thies

ACT:vr
Attachment



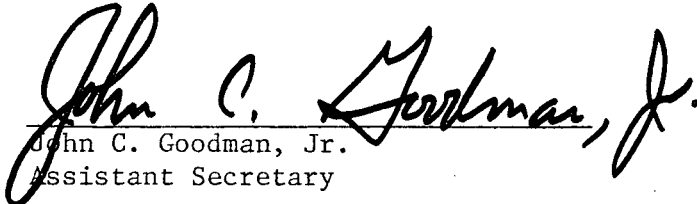
Mr. Angelo Giambusso
Page 2
August 30, 1974

A. C. THIES, being duly sworn, states that he is Senior Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Atomic Energy Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.



A. C. Thies
A. C. Thies, Senior Vice President

ATTEST:


John C. Goodman, Jr.
Assistant Secretary

Subscribed and sworn to before me this 30th day of August, 1974.


Anne E. Leach
Notary Public

My Commission Expires:

September 16, 1974

4.17 STEAM GENERATOR TUBING SURVEILLANCE

Applicability

Applies to the surveillance of tubing of each steam generator.

Objective

To define the in-service surveillance program for steam generator tubing.

Specification

4.17.1 Baseline Inspection

The first steam generator tubing inspection performed according to Specifications 4.17.2 and 4.17.3 shall be considered as constituting the baseline condition for subsequent inspections.

4.17.2 Examination Methods

In-service inspection of steam generator tubing shall include non-destructive examination by eddy-current testing or other equivalent techniques. The inspection equipment shall provide a sensitivity that will detect defects with a penetration of 20 percent or more of the minimum allowable as-manufactured tube wall thickness.

4.17.3 Selection and Testing

Selection and testing of tubes from each steam generator shall be made on the basis of the following:

- a. No fewer than 3 percent of the total number of steam generator tubes shall undergo examination during each in-service inspection.
- b. Tubes for the baseline inspection shall be selected on a random basis.
- c. Every inspection subsequent to the baseline inspection shall include all tubes which previously had detectable wall penetration (greater than 20 percent and not including plugged tubes), and shall also consider tubes and those areas where design and experience have indicated potential problems.

If the inspection indicates that more than 10 percent of the inspected tubes have detectable wall penetration or that one or more of the inspected tubes have an indication of an unacceptable defect, an additional 3 percent of the tubes shall be inspected, concentrating on tubes in those areas of the tube sheet array where tubes with defects were found. If the inspection of these additional tubes indicates that more than 10 percent of the inspected tubes have detectable wall penetration or one or more of the inspected tubes has an indication of an unacceptable defect, additional tubes (no less than 3 percent of the total tubes in the steam generator) in the area of the defect shall be inspected.

4.17.4 Inspection Intervals

- a. In-service inspection of steam generator tubing shall be performed during each refueling outage, except that the first scheduled inspection after the baseline inspection need not be performed until the second refueling outage following the baseline inspection, provided the baseline inspection results did not indicate any tubes with detectable wall penetration.
- b. If two consecutive inspections, excluding the baseline inspection, result in no additional tubes with detectable wall penetration and no significant (greater than 5 percent) further penetration of tubes which previously had indications, then inspections shall be performed three times every 10 years at approximately equal intervals, coinciding with refueling outages.

4.17.5 Acceptance Criteria

- a. If less than 10 percent of the total tubes inspected have detectable wall penetration and no more than three tubes have unacceptable defects, operation may resume after required corrective measures have been taken.
- b. If more than 10 percent of the total tubes inspected have detectable wall penetration or more than three of the tubes inspected have unacceptable defects, operation may resume after required corrective measures have been taken, and the situation and remedial action shall be reported to the Atomic Energy Commission.

4.17.6 Corrective Measures

All tubes with unacceptable defects shall be plugged.

Bases

The program of periodic in-service inspection of steam generators provides the means of monitoring the integrity of the tubing and to maintain surveillance in the event there is evidence of mechanical damage or progressive deterioration due to design, manufacturing errors, or operating conditions. In-service inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures may be taken. In-service inspection includes non-destructive examination using a suitable eddy-current inspection system (or other equivalent techniques), capable of locating and identifying defects due to stress corrosion cracking, mechanical damage, chemical wastage, or other causes.

An unacceptable defect is defined as one which would result in not satisfying the calculated acceptable minimum tube wall thickness that can sustain a loss-of-coolant accident in combination with a safe shutdown earthquake.

The baseline inspection is performed during the first refueling outage. In addition to establishing a basis for comparison for subsequent inspections, the baseline inspection also allows evaluation of the steam generator tubing performance during startup testing. If no problem are found, the next inspection is performed during the third refueling outage when long-term effects should become detectable.

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