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CONTROL NO: 2971

FILE: _____

FROM: Duke Power Company Charlotte, N.C. 28201 A.C. Thies		DATE OF DOC 3-12-75	DATE REC'D 3-18-75	LTR XX	TWX	RPT	OTHER
TO: Mr. A. Giambusso		ORIG 3 signed	CC 37	OTHER	SENT AEC PDR <u>XX</u> SENT LOCAL PDR <u>XX</u>		
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 40	DOCKET NO: <u>50-269/270/287</u>		
DESCRIPTION: Ltr notarized 3-12-75 requesting that Tech Spec 1.2.8 be deleted & trans the following:				ENCLOSURES: Replacement page 1-2 to Oconee Tech Specs.... (40 cys encl rec'd)			
PLANT NAME: Oconee Units 1-2-3				Do Not Remove ACKNOWLEDGED			

FOR ACTION/INFORMATION

DHL 3-19-75

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<u>FILE & REGION (2)</u>	BENAROYA	<u>M. AU</u>	V. WILSON (L)	WIGGINTON
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1 - ASLB	NEWMARK/BLUME/AGBABIAN	1 - AGMED (RUTH GUSSMAN) Rm B-127 GT
<u>1</u> - Newton Anderson		1 - J. D. RUNKLES, Rm E-201 GT
<u>14</u> ACRS XXXXXXXX SENT TO LIC. ASST. SHEPPARD 3-19-75		

Regulatory Docket File

DUKE POWER COMPANY

POWER BUILDING

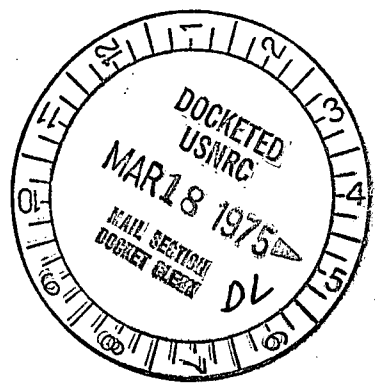
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

March 12, 1975

Mr. Angelo Giambusso, Director
Division of Reactor Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555



Re: Oconee Nuclear Station
Docket Nos. 10-269 -270, -287

Dear Mr. Giambusso:

Section 1.2.8 of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, -47, and 55 defines the Refueling Period as the time between normal refuelings of the reactor, not to exceed 18 months without prior NRC approval. The basis for requiring prior NRC approval before exceeding an 18 month interval was that many surveillance tests were specified to be performed during a refueling outage. Recent amendments to the Technical Specifications have revised these surveillance tests to a calendar basis with the exception of those which are directly associated with refueling.

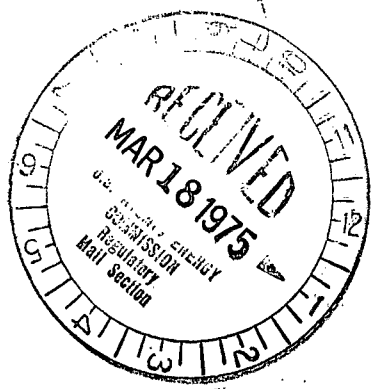
Pursuant to 10 CFR 50.90, it is requested that Technical Specification 1.2.8 be deleted. The attached proposed replacement page shows this Technical Specification deleted and the remaining specifications renumbered.

Very truly yours,

A. C. Thies

ACT:vr

Attachment




Mr. Angelo Giambusso
Page 2
March 12, 1975

A. C. THIES, being duly sworn, states that he is Senior Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Atomic Energy Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

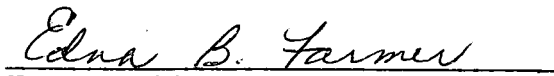


A. C. Thies, Senior Vice President

ATTEST:


John C. Goodman, Jr.
Assistant Secretary

Subscribed and sworn to before me this 12th day of March, 1975.


Edna B. Farmer
Notary Public

My Commission Expires:

October 24, 1977

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1.2.7 Refueling Operation

An operation involving a change in core geometry by manipulation of fuel or control rods when the reactor vessel head is removed.

1.2.8 Startup

The reactor shall be considered in the startup mode when the shutdown margin is reduced with the intent of going critical.

1.3 OPERABLE

A component or system is operable when it is capable of performing its intended function within the required range. The component or system shall be considered to have this capability when: (1) it satisfies the limiting conditions for operation defined in Specification 3, and (2) it has been tested periodically in accordance with Specification 4 and has met its performance requirements.

1.4 PROTECTIVE INSTRUMENTATION LOGIC

1.4.1 Instrument Channel

An instrument channel is the combination of sensor, wires, amplifiers and output devices which are connected for the purpose of measuring the value of a process variable for the purpose of observation, control and/or protection. An instrument channel may be either analog or digital in nature.

1.4.2 Reactor Protective System

The reactor protective system is shown in Figures 7-1 and 7-6 of the FSAR. It is that combination of protective channels and associated circuitry which forms the automatic system that protects the reactor by control rod trip. It includes the four protective channels, their associated instrument channel inputs, manual trip switch, all rod drive protective trip breakers and activating relays or coils.

1.4.3 Protective Channel

A protective channel as shown in Figure 7-1 of the FSAR (one of three or one of four independent channels, complete with sensors, sensor power supply units, amplifiers and bistable modules provided for every reactor protective safety parameter) is a combination of instrument channels forming a single digital output to the protective system's coincidence logic. It includes a shutdown bypass circuit, a protective channel bypass circuit and reactor trip module and provision for insertion of a dummy bistable.