

FROM: Duke Power Company
Charlotte, N.C. 28201
A.C. Thies

DATE OF DOCUMENT: Mar. 8, 1972
DATE RECEIVED: Mar. 13, 1972
NO.: 54

LTR. MEMO: REPORT: OTHER:

TO: Mr. R.C. DeYoung

ORIG.: 3 signed
CC: OTHER:

ACTION NECESSARY CONCURRENCE DATE ANSWERED:
NO ACTION NECESSARY COMMENT BY:

CLASSIF.: U POST OFFICE REG. NO:

FILE CODE: 50-269 50-270 50-287

DESCRIPTION: (Must Be Unclassified)
Ltr re our 2-10-72 ltr..furnishing addl
info on Inservice Inspection Program for
Oconee Station.....

REFERRED TO	DATE	RECEIVED BY	DATE
Schwencer w/4 cys for ACTION	3-14-72		

ENCLOSURES:

DISTRIBUTION:
Neg File Cy (3)
AEC PDR (3)
Compliance (2)
OGC-Rm-P-506-A
DeYoung
Case/Maccary
Hanauer
Skovholt

REMARKS:
1 CY LOCAL PDR WALHALLA, S.C.

P. Collins
Schroeder
T.R. Wilson
Denton
Klecker

**DO NOT REMOVE
ACKNOWLEDGED**

1234 DL

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

March 8, 1972



United States Atomic Energy Commission
Division of Reactor Licensing
7920 Norfolk Avenue
Bethesda, Maryland

Attention: Mr. R. C. DeYoung, Assistant Director
for Pressurized Water Reactors

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, and -287

Gentlemen:

Please reference your letter of February 10, 1972. The inspections that will be performed on the reactor coolant system at Oconee do comply with the intent of the "Recommended PWR Inservice Inspection Program for Detection of Effects of Reactor Coolant Leakage." The methods normally employed to detect coolant leakage are described in Section 3.1.6 of the Oconee Technical Specifications and provide an indication of the potential for prolonged contact of borated fluid with ferritic steel. Continuous surveillance of coolant inventory, activity monitoring, sump level monitoring, and physical inspection by operating personnel will identify coolant leakage during normal operation. A physical inspection of the reactor coolant system during each refueling shutdown will identify boric acid crystalline deposits from minute leakage during operation. Sources of leakage shall be corrected and ferritic steel components of the reactor coolant pressure boundary exposed to boric acid residue will be examined for evidence of corrosion. Prior to startup following each refueling outage, the reactor coolant system will be examined under not less than normal operating pressure to ensure leak tight integrity during operation as described in Section 4.3 of the Oconee Technical Specifications.

The program outlined above complies with our interpretation of the AEC's "Recommended PWR Inservice Inspection Program for Detection of Effects of Reactor Coolant Leakage." It is our interpretation that paragraph B(1)a of the recommended program describes corrective measures applied to leakage that cannot be eliminated and does not require provisions for collection and disposal of potential leakage as preventative measures.

United States Atomic Energy Commission
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With the above clarification, we will comply with the requirements of the program recommended as a part of the Oconee Inservice Inspection Program.

Sincerely,

A handwritten signature in cursive script that reads "A. C. Thies".

A. C. Thies

ACT:vr