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FROM	DATE OF DOC	DATE REC'D	LTR	TWX	RPT	OTHER
Duke Power Company Charlotte, N.C. 28201 Mr. A.C. Thies	11-12-74	11-15-74	X			

TO:	ORIG	CC	OTHER	SENT AEC PDR	SENT LOCAL PDR
A. Giambusso	3 signed			XXX	XXX

CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:
	XXX		XXX	40	50-269/270/287

DESCRIPTION:

Ltr requesting an amdt to the OL... trans the following...

*Denotes Ltr only

PLANT NAME: Oconee

ENCLOSURES:

Proposed change to the OL consist of rev pgs to the tech specs.....

(40 cys encl rec'd)

ACKNOWLEDGED

DO NOT REMOVE

FOR ACTION/INFORMATION 11-18-74 JB

BUTLER (L) W/ Copies	SCHWENCER (L) W/ Copies	ZIEMANN (L) W/ Copies	REGAN (E) W/ Copies
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KNIEL (L) W/ Copies	✓PURPLE (L) W/ Copies	YOUNGBLOOD (E) W/ Copies	W/ Copies

INTERNAL DISTRIBUTION

✓REG FILE	TECH REVIEW	DENTON	LIC ASST	A/T IND
✓AEC PDR		GRIMES		BRAITMAN
✓OGC, ROOM P-506A	SCHROEDER	GAMMILL	DIGGS (L)	SALTZMAN
✓MUNTZING/STAFF	✓MACCARY	KASTNER	GEARIN (L)	B. HURT
✓CASE	KNIGHT	BALLARD	GOULBOURNE (L)	
GIAMBUSSO	PAWLICKI	SPANGLER	KREUTZER (E)	PLANS
BOYD	SHAO		LEE (L)	MCDONALD
MOORE (L) (BWR)	✓STELLO	ENVIRO	MAIGRET (L)	CHAPMAN
DEYOUNG (L) (PWR)	HOUSTON	MULLER	REED (E)	✓DUBE w/input
SKOVHOLT (L)	NOVAK	DICKER	SERVICE (L)	✓E. COUPE
✓ *GOLLER (L)	ROSS	KNIGHTON	✓SHEPPARD (L)	✓Schemel
P. COLLINS	IPPOLITO	YOUNGBLOOD	SLATER (E)	D. THOMPSON (2)
DENISE	✓TEDESCO	REGAN	SMITH (L)	KLECKER
✓REG OPR	LONG	✓PROJECT LDR	TEETS (L)	EISENHUT
FILE & REGION (3)	LAINAS	Scalletti	WILLIAMS (E)	
MORRIS	BENAROYA	HARLESS	WILSON (L)	
STEELE	VOLIMER			

EXTERNAL DISTRIBUTION

Appl. H

✓1 – LOCAL PDR Walhalla, SC.

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1 – ASLB

1 – Newton Anderson

✓16 – ACRS ~~HOLDING~~ Sent to Sheppard 11-18-74

1 – ASLBP(E/W Bldg, Rm 529)

1 – W. PENNINGTON, Rm E-201 GT

1 – B&M SWINEBROAD, Rm E-201 GT

1 – CONSULTANTS NEWMARK/BLUME/AGBABIAN

1 – PDR-SAN/LA/NY

1 – BROOKHAVEN NAT LAB

1 – G. ULRIKSON, ORNL

1 – AGMED (RUTH GUSSMAN) Rm B-127 GT

1 – R. D. MUELLER, Rm E-201 GT

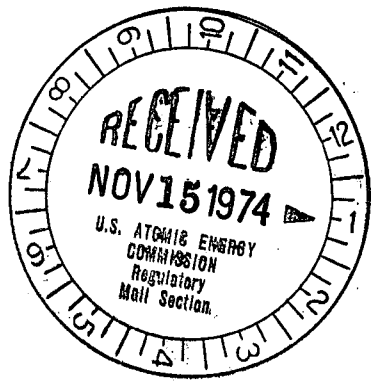
Regulatory Docket File

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. BOX 2178

November 12, 1974



Mr. Angelo Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, and -287

Dear Mr. Giambusso:

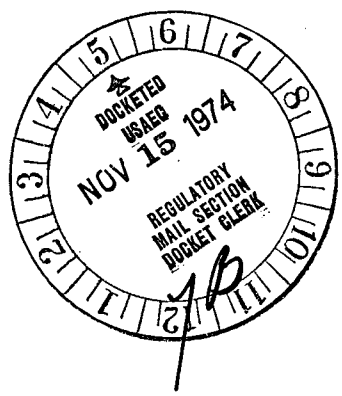
Pursuant to 10CFR50.90, a change is requested to Oconee Nuclear Station Technical Specification 4.2, "Reactor Coolant System Surveillance. The purpose of this revision is to modify the reporting requirements in Specification 4.2.8 to be consistent with Specification 6.6.1.6. This will require a report of the test results of the reactor vessel specimen withdrawal to be forwarded to the AEC in accordance with 10CFR50, Appendix H. A proposed replacement page for the Technical Specifications, which delineates the desired change, is attached.

Very truly yours,

A. C. Thies

A. C. Thies

ACT:vr



11693

A. C. THIES, being duly sworn, states that he is Senior Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Atomic Energy Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

A. C. Thies, Senior Vice President

ATTEST:

John C. Goodman, Jr.
Assistant Secretary

Subscribed and sworn to before me this 12th day of November, 1974.

Notary Public

My Commission Expires:

October 24, 1977

4.2.3 The structural integrity of the Reactor Coolant System boundary shall be maintained at the level required by the original acceptance standards throughout the life of the station. Any evidence, as a result of the tests outlined in Table IS-261 of Section XI of the code, that defects have developed or grown, shall be investigated, including evaluation of comparable areas of the Reactor Coolant System.

4.2.4 To assure the structural integrity of the reactor internals throughout the life of the unit, the two sets of main internals bolts (connecting the core barrel to the core support shield and to the lower grid cylinder) shall remain in place and under tension. This will be verified by visual inspection to determine that the welded bolt locking caps remain in place. All locking caps will be inspected after hot functional testing and whenever the internals are removed from the vessel during a refueling or maintenance shutdown. The core barrel to core support shield caps will be inspected each refueling shutdown.

4.2.5 Sufficient records of each inspection shall be kept to allow comparison and evaluation of future inspections.

4.2.6 The inservice inspection program shall be reviewed at the end of five years to consider incorporation of new inspection techniques and equipment which have been proved practical and the conclusions of this review and evaluation shall be discussed with the AEC/DOL.

4.2.7 At approximately three-year intervals, the bore and keyway of each reactor coolant pump flywheel shall be subjected to an in-place, volumetric examination. Whenever maintenance or repair activities necessitate flywheel removal, a surface examination of exposed surfaces and a complete volumetric examination shall be performed, if the interval measured from the previous such inspections is greater than 6 2/3 years.

4.2.8 For Unit 1 and Unit 2, a B Type vessel specimen capsule shall be withdrawn after one year of operation and an A Type capsule shall be withdrawn after 11, 17, and 22 years of operation. The withdrawal schedules may be modified to coincide with those refueling outages or unit shutdowns most closely approaching the withdrawal schedule. Specimens thus withdrawn shall be tested in accordance with ASTM-E-185-70 pursuant to 10CFR50, Appendix H. For Unit 3, a B Type vessel specimen capsule shall be withdrawn after one year of operation and an A Type capsule shall be withdrawn after 7, 14, and 17 years of operation. The withdrawal schedules may be modified to coincide with those refueling outages or unit shutdowns most closely approaching the withdrawal schedule. Specimens thus withdrawn shall be tested in accordance with ASTM-E-185-72 pursuant to 10CFR50, Appendix H.