



Nebraska Public Power District

Always there when you need us

NLS2016005
January 12, 2016

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555-0001

Subject: Correction to License Amendment Request to Revise Technical Specifications to Add Residual Heat Removal System Containment Spray Function
Cooper Nuclear Station, Docket No. 50-298, DPR-46

Reference: Letter from Oscar A. Limpias, Nebraska Public Power District, to the U.S. Nuclear Regulatory Commission, dated January 15, 2015, "License Amendment Request to Revise Technical Specifications to Add Residual Heat Removal System Containment Spray Function" (ML15021A127)

Dear Sir or Madam:

The purpose of this letter is for the Nebraska Public Power District to correct an error on a Technical Specifications (TS) page submitted with the proposed referenced amendment to add new section, TS 3.6.1.9, Residual Heat Removal (RHR) Containment Spray, and add related drywell pressure switches to TS 3.3.5.1, Emergency Core Cooling System (ECCS) Instrumentation.

The proposed license amendment request contains an error in the section title at the top of TS page 3.6-25 included in Attachments 2 and 3 of the submittal. The section title as submitted is 3.6 CONTAINMENT SPRAY and needs to be corrected to state 3.6 CONTAINMENT SYSTEMS. Attachment 1 to this letter contains a corrected TS page 3.6-25.

The Table of Contents is affected by the proposed TS change but was not included in the referenced amendment request. Attachment 2 to this letter contains the revised Table of Contents.

The editorial changes submitted in this letter neither affect the technical or regulatory analysis included in the license amendment request, nor do they affect the results of the no significant hazards evaluation.

This letter does not contain any new regulatory commitments.

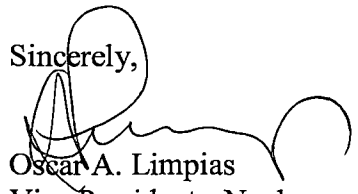
If you have any questions concerning this matter, please contact Jim Shaw, Licensing Manager, at (402) 825-2788.

ADD
NRR

NLS2016005

Page 2 of 2

Sincerely,



Oscar A. Limpas
Vice President - Nuclear
and Chief Nuclear Officer

/dv

- Attachments:
1. Corrected Technical Specifications Page 3.6-25
 2. Revised Technical Specifications Table of Contents

cc: Regional Administrator w/ attachments
USNRC - Region IV

Cooper Project Manager w/ attachments
USNRC - NRR Plant Licensing Branch IV-2

Senior Resident Inspector w/ attachments
USNRC - CNS

NPG Distribution w/o attachments

CNS Records w/ attachments

NLS2016005
Attachment 1
Page 1 of 2

Attachment 1

Corrected Technical Specifications Page 3.6-25

Cooper Nuclear Station, Docket No. 50-298, DPR-46

3.6 CONTAINMENT SYSTEMS

3.6.1.9 Residual Heat Removal (RHR) Containment Spray

LCO 3.6.1.9 Two RHR containment spray subsystems shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One RHR containment spray subsystem inoperable.	A.1 Restore RHR containment spray subsystem to OPERABLE status.	7 days
B. Two RHR containment spray subsystems inoperable.	B.1 Restore one RHR containment spray subsystem to OPERABLE status.	8 hours
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3.	12 hours
	<u>AND</u> C.2 Be in MODE 4.	36 hours

Attachment 2

Revised Technical Specifications Table of Contents

Cooper Nuclear Station, Docket No. 50-298, DPR-46

TABLE OF CONTENTS

1.0	USE AND APPLICATION	1.1-1
1.1	Definitions	1.1-1
1.2	Logical Connectors	1.2-1
1.3	Completion Times	1.3-1
1.4	Frequency.....	1.4-1
2.0	SAFETY LIMITS (SLs)	2.0-1
2.1	SLs	2.0-1
3.0	LIMITING CONDITION FOR OPERATION (LCO) APPLICABILITY	3.0-1
3.0	SURVEILLANCE REQUIREMENT (SR) APPLICABILITY	3.0-4
3.1	REACTIVITY CONTROL SYSTEMS	3.1-1
3.1.1	SHUTDOWN MARGIN (SDM)	3.1-1
3.1.2	Reactivity Anomalies.....	3.1-5
3.1.3	Control Rod OPERABILITY	3.1-7
3.1.4	Control Rod Scram Times.....	3.1-12
3.1.5	Control Rod Scram Accumulators	3.1-15
3.1.6	Rod Pattern Control	3.1-18
3.1.7	Standby Liquid Control (SLC) System.....	3.1-20
3.1.8	Scram Discharge Volume (SDV) Vent and Drain Valves.....	3.1-25
3.2	POWER DISTRIBUTION LIMITS.....	3.2-1
3.2.1	AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR).....	3.2-1
3.2.2	MINIMUM CRITICAL POWER RATIO (MCPR).....	3.2-2
3.3	INSTRUMENTATION	3.3-1
3.3.1.1	Reactor Protection System (RPS) Instrumentation.....	3.3-1
3.3.1.2	Source Range Monitor (SRM) Instrumentation.....	3.3-9
3.3.2.1	Control Rod Block Instrumentation.....	3.3-14
3.3.2.2	Feedwater and Main Turbine High Water Level Trip Instrumentation	3.3-20
3.3.3.1	Post Accident Monitoring (PAM) Instrumentation	3.3-22
3.3.3.2	Alternate Shutdown System.....	3.3-26
3.3.4.1	Anticipated Transient Without Scram Recirculation Pump Trip (ATWS-RPT) Instrumentation	3.3-28
3.3.5.1	Emergency Core Cooling System (ECCS) Instrumentation	3.3-31
3.3.5.2	Reactor Core Isolation Cooling (RCIC) System Instrumentation	3.3-43
3.3.6.1	Primary Containment Isolation Instrumentation.....	3.3-47
3.3.6.2	Secondary Containment Isolation Instrumentation	3.3-54
3.3.6.3	Low-Low Set (LLS) Instrumentation	3.3-58

TABLE OF CONTENTS (continued)

3.3	INSTRUMENTATION (continued)	
3.3.7.1	Control Room Emergency Filter (CREF) System Instrumentation.....	3.3-61
3.3.8.1	Loss of Power (LOP) Instrumentation	3.3-64
3.3.8.2	Reactor Protection System (RPS) Electric Power Monitoring	3.3-67
3.4	REACTOR COOLANT SYSTEM (RCS).....	3.4-1
3.4.1	Recirculation Loops Operating	3.4-1
3.4.2	Jet Pumps.....	3.4-4
3.4.3	Safety/Relief Valves (SRVs) and Safety Valves (SVs)	3.4-6
3.4.4	RCS Operational LEAKAGE	3.4-8
3.4.5	RCS Leakage Detection Instrumentation	3.4-10
3.4.6	RCS Specific Activity	3.4-12
3.4.7	Residual Heat Removal (RHR) Shutdown Cooling System - Hot Shutdown.....	3.4-14
3.4.8	Residual Heat Removal (RHR) Shutdown Cooling System - Cold Shutdown	3.4-17
3.4.9	RCS Pressure and Temperature (P/T) Limits.....	3.4-19
3.4.10	Reactor Steam Dome Pressure	3.4-26
3.5	EMERGENCY CORE COOLING SYSTEMS (ECCS) AND REACTOR CORE ISOLATION COOLING (RCIC) SYSTEM.....	3.5-1
3.5.1	ECCS - Operating	3.5-1
3.5.2	ECCS - Shutdown.....	3.5-7
3.5.3	RCIC System	3.5-11
3.6	CONTAINMENT SYSTEMS	3.6-1
3.6.1.1	Primary Containment	3.6-1
3.6.1.2	Primary Containment Air Lock.....	3.6-3
3.6.1.3	Primary Containment Isolation Valves (PCIVs)	3.6-8
3.6.1.4	Drywell Pressure.....	3.6-16
3.6.1.5	Drywell Air Temperature	3.6-17
3.6.1.6	Low-Low Set (LLS) Valves.....	3.6-18
3.6.1.7	Reactor Building-to-Suppression Chamber Vacuum Breakers.....	3.6-20
3.6.1.8	Suppression-Chamber-to-Drywell Vacuum Breakers	3.6-23
3.6.1.9	Residual Heat Removal (RHR) Containment Spray	3.6-25
3.6.2.1	Suppression Pool Average Temperature	3.6-27
3.6.2.2	Suppression Pool Water Level	3.6-30
3.6.2.3	Residual Heat Removal (RHR) Suppression Pool Cooling.....	3.6-31
3.6.3.1	Primary Containment Oxygen Concentration	3.6-33
3.6.4.1	Secondary Containment.....	3.6-34
3.6.4.2	Secondary Containment Isolation Valves (SCIVs).....	3.6-36
3.6.4.3	Standby Gas Treatment (SGT) System.....	3.6-40

TABLE OF CONTENTS (continued)

3.7	PLANT SYSTEMS	3.7-1
3.7.1	Residual Heat Removal Service Water Booster (RHRSWB) System	3.7-1
3.7.2	Service Water (SW) System and Ultimate Heat Sink (UHS).....	3.7-3
3.7.3	Reactor Equipment Cooling (REC) System.....	3.7-6
3.7.4	Control Room Emergency Filter (CREF) System	3.7-8
3.7.5	Air Ejector Offgas.....	3.7-11
3.7.6	Spent Fuel Storage Pool Water Level	3.7-13
3.7.7	The Main Turbine Bypass System	3.7-14
3.8	ELECTRICAL POWER SYSTEMS	3.8-1
3.8.1	AC Sources - Operating	3.8-1
3.8.2	AC Sources - Shutdown.....	3.8-10
3.8.3	Diesel Fuel Oil, Lube Oil, and Starting Air	3.8-13
3.8.4	DC Sources - Operating.....	3.8-16
3.8.5	DC Sources - Shutdown.....	3.8-20
3.8.6	Battery Cell Parameters	3.8-22
3.8.7	Distribution Systems - Operating.....	3.8-26
3.8.8	Distribution Systems - Shutdown	3.8-29
3.9	REFUELING OPERATIONS	3.9-1
3.9.1	Refueling Equipment Interlocks.....	3.9-1
3.9.2	Refuel Position One-Rod-Out Interlock	3.9-3
3.9.3	Control Rod Position.....	3.9-4
3.9.4	Control Rod Position Indication.....	3.9-5
3.9.5	Control Rod OPERABILITY - Refueling	3.9-7
3.9.6	Reactor Pressure Vessel (RPV) Water Level	3.9-8
3.9.7	Residual Heat Removal (RHR) - High Water Level	3.9-9
3.9.8	Residual Heat Removal (RHR) - Low Water Level	3.9-12
3.10	SPECIAL OPERATIONS	3.10.1
3.10.1	Inservice Leak and Hydrostatic Testing Operation	3.10-1
3.10.2	Reactor Mode Switch Interlock Testing	3.10-4
3.10.3	Single Control Rod Withdrawal - Hot Shutdown	3.10-6
3.10.4	Single Control Rod Withdrawal - Cold Shutdown	3.10.9
3.10.5	Single Control Rod Drive (CRD) Removal - Refueling.....	3.10-13
3.10.6	Multiple Control Rod Withdrawal - Refueling	3.10-16
3.10.7	Control Rod Testing - Operating	3.10-18
3.10.8	SHUTDOWN MARGIN (SDM) Test - Refueling	3.10-20
4.0	DESIGN FEATURES	4.0-1
4.1	Site Location	4.0-1
4.2	Reactor Core	4.0-1
4.3	Fuel Storage	4.0-2

TABLE OF CONTENTS (continued)

5.0	ADMINISTRATION CONTROLS	5.0-1
5.1	Responsibility.....	5.0-1
5.2	Organization	5.0-2
5.3	Unit Staff Qualifications.....	5.0-4
5.4	Procedures	5.0-5
5.5	Programs and Manuals.....	5.0-6
5.6	Reporting Requirements	5.0-18
5.7	High Radiation Area.....	5.0-21
