

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO:
Mr. Benard C. Rusche

FROM:
Duke Power Company
Charlotte, North Carolina
Mr. William O. Parker, Jr.

DATE OF DOCUMENT
9/14/76

DATE RECEIVED
9/20/76

LETTER
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DESCRIPTION

Ltr. re our 6/8/76 ltr. and their 8/13/76 ltr....concerning their response to questions relative to the reactor vessel support system.

(1-P)

PLANT NAME:
Oconee 1-2-3

DISTRIBUTION FOR REACTOR VESSEL SUPPORT INFO FOR OPERATING REACTORS PER MR. TRAMMELL 7-12-76

ENCLOSURE

ACKNOWLEDGED
DO NOT REMOVE

SAFETY FOR ACTION/INFORMATION 9/23/76 RJL

<input checked="" type="checkbox"/> ASSIGNED AD:	Goller		
<input checked="" type="checkbox"/> BRANCH CHIEF:	Schwencer		
<input checked="" type="checkbox"/> PROJECT MANAGER:	Zech		
<input checked="" type="checkbox"/> LIC. ASST:	Sheppard		

INTERNAL DISTRIBUTION

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<input checked="" type="checkbox"/> SHAO			
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<input checked="" type="checkbox"/> R. BOSNAK			
<input checked="" type="checkbox"/> V. NOONAN			

EXTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> LPDR: Walhalla, S.C.			
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CONTROL NUMBER
9530

DUKE POWER COMPANY

POWER BUILDING

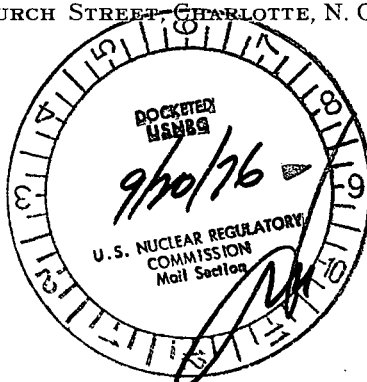
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

REGULATORY DOCKET FILE COPY

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

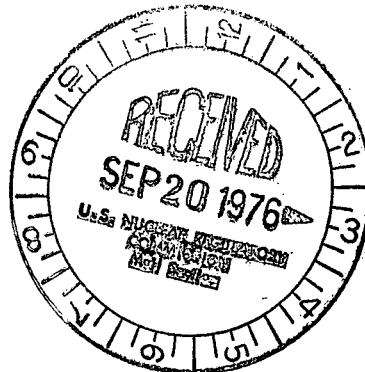
September 14, 1976



Mr. Benard C. Rusche
Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. A. Schwencer, Chief
Operating Reactors Branch #1

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

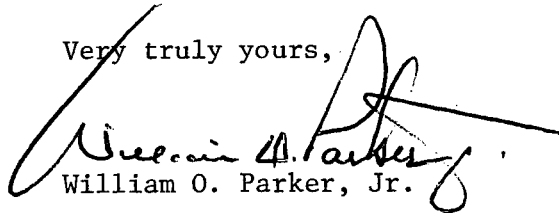


Dear Mr. Rusche:

Your letter of June 8, 1976 requested that a schedule be submitted concerning our response to questions relative to the Oconee Nuclear Station reactor vessel support system. Subsequently, in our August 13, 1976 letter, it was stated that a users group of utilities that have Babcock and Wilcox 177 FA Nuclear Steam Supply Systems had been formed to develop plans and schedules for resolution of this concern. Duke Power Company, in conjunction with this users group and several consultants, has reviewed the probability study which was developed by the Combustion Engineering users group and presented to the NRC staff. Based on this study, and other information currently available, we believe the probability of the occurrence of the particular incident of concern is acceptably remote and no further analysis is necessary. In order to evaluate and quantify this probability, we are having a study performed of the probability of a failure of the reactor coolant system in the annular region between the reactor vessel and the cavity shield wall.

We expect this study to be completed by mid December, 1976, and will at that time discuss the results with your staff.

Very truly yours,


William O. Parker, Jr.

MST:vr

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