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FROM: Duke Power Company Charlotte, N.C. 28201 A.C. Thies	DATE OF DOC 4-16-75	DATE REC'D 4-19-75	LTR XX	TWX	RPT	OTHER
TO: Mr. R.A. Purple	ORIG 1 signed	CC / OTHER	SENT AEC PDR XX SENT LOCAL PDR XX			
CLASS / UNCLASS / PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-269/270/287			

DESCRIPTION: Ltr re our 4-3-75 ltr.... furnishing addl info re definition of refueling period for Oconee Tech Specs...trans the following:

ENCLOSURES: Surveillance Items Required During Refueling Outage....

(1 cy encl rec'd)

PLANT NAME: Oconee Units 1-2-3

FOR ACTION/INFORMATION DHL 4-23-75

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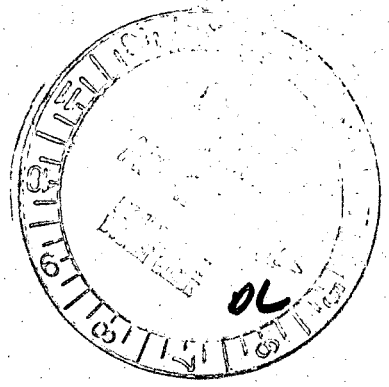
A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. BOX 2178

April 16, 1975



Mr. R. A. Purple, Chief
Operating Reactors Branch 1
Division of Reactor Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Purple:

Your letter of April 3, 1975 requested additional information necessary to complete your review of my March 12, 1975 request to delete the definition of refueling period (Section 1.2.8) for Oconee Nuclear Station Technical Specifications. Attached is the listing of all Oconee Nuclear Station surveillance requirements which are specified for performance prior to, during, or after a refueling shutdown with an explanation of why they can only be/or should be performed at this time.

The one-hour discharge test of the 125 volt DC batteries required by Technical Specification 4.6.6.c, currently required during a refueling outage, does not require a refueling outage for its completion. A change to Oconee Nuclear Station Technical Specifications is hereby requested which will make this an annual surveillance item.

Very truly yours,

A. C. Thies

A. C. Thies

ACT:vr

Attachment

4230

SURVEILLANCE ITEMS REQUIRED DURING REFUELING OUTAGE

Technical Specification 4.1.2

4-16-75

1. Functional Tests and Refueling System Interlocks
2. Functional Test of Spent Fuel Cooling System

RESPONSE

These two items are scheduled prior to refueling. The intention is to test these systems immediately prior to their use.

Technical Specification 4.6.3

During each refueling outage for the affected unit, a simulated emergency transfer from the 4160 volt main feeder buses to the startup transformer (i.e., CT1, CT2, or CT3) to the 4160 volt standby buses shall be made to verify proper operation.

RESPONSE

The performance of this test requires a complete unit blackout for approximately four hours. Therefore, in order to ensure adequate decay heat removal, the test must be performed at the end of a refueling interval when decay heat generation is a minimum. During the test, the reactor vessel head is removed and the fuel transfer canal is filled to provide heat transfer. It is concluded that this test is not feasible at times other than a refueling outage.

Technical Specification 4.7.1

The control rod trip insertion time shall be measured for each control rod at either full flow or no flow conditions following each refueling outage prior to return to power.

RESPONSE

This test is performed following refueling to assure the proper installation of control rods after the installation of the reactor vessel head.