)05/11/1 REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS) DISTRIBUTION FOR INCOMING MATERIAL 50 - 270REC: OREILLY J P ORG: PARKER W O DOCDATE: 05/05/78 NRC DUKE PWR DATE RCVD: 05/10/78 DOCTYPE: LETTER NOTARIZED: NO COPIES RECEIVED SUBJECT: LTR 1 ENCL 1 FORWARDING LICENSEE EVENT REPT (R0 50-270/78-006) ON 04/07/78 CONCERNING THE RC LEAK RATE EXCEEDED 1 GPM... LEAK WAS PRIMARILY A PACKING LEAK ON 2RC-3, THE SPRAY CONTROL BLOCK VALVE, RESULTING FROM BLOWN PACKING ... W/ATT. PLANT NAME: OCONEE - UNIT 2 REVIEWER INITIAL: X. IM DISTRIBUTOR INITIAL: NOTES: 1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS INCIDENT REPORTS (DISTRIBUTION CODE A002) RETURNU/4 ENCL BR CHIEF FOR ACTION: INTERNAL: REG FILE**W/ENC NRC PDR**W/ENCL I & E**W/Z ENCL MIPC**W/3 ENCL SCHROEDER/IPPOLITO**W/ENCL HOUSTON**W/ENCL NOVAK/CHECK**W/ENCL EEB**W/ENCL KNIGHT**W/ENCL BUTLER**W/ENCL HANAUER***W/ENCL TEDESCO**W/ENCL EISENHUT**W/ENCL BAER**W/ENCL SHAO**W/ENCL VOLLMER/BUNCH**W/ENCL KREGER/J. COLLINS**W/ENCL ROSA**W/ENCL K SEYFRIT/IE**W/ENCL EXTERNAL: LPDR'S WALHALLA, SC**W/ENCL TIC**W/ENCL NSIC**W/ENCL ACRS CAT B**W/16 ENCL COPIES NOT SUBMITTED PER REGULATORY GUIDE 10.1 DISTRIBUTION: LTR 45 ENCL 45 CONTROL NBR: 781310011 SIZE: 1P+1P+1P ***** THE END

DUKE POWER COMPANY POWER BUILDING REGULATORY DOCKET FILE COPY

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TECEPHONE: AREA 704 373-4083

WILLIAM O. PARKER, JR. VICE PRESIDENT STEAM PRODUCTION



Mr. James P. O'Reilly, Director U. S. Nuclear Regulatory Commission Suite 1217 230 Peachtree Street, Northwest Atlanta, GA 30303

RE: Oconee Unit 2 Docket No. 50-270

Dear Mr. O'Reilly:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-270/78-6.

Very truly yours,

am O. Parker by WAH William O. Parker, Jr.

KRW/rpc

Attachment

cc: Director, Office of Management Information and Program Control DUKE POWER COMPANY OCONEE UNIT 2

Report Number: RO-270/78-6

Report Date: May 5, 1978

Occurrence Date: April 7, 1978

Facility: Oconee Unit 2, Seneca, South Carolina

Identification of Occurrence: Reactor Coolant Leakage in Excess of 1 gpm

Conditions Prior to Occurrence: 100% Full Power

Description of Occurrence:

At 1000, on April 6, 1978, it was determined that the reactor coolant leakage rate exceeded 1 gpm and an investigation was commenced. By 1500, personnel had entered the Reactor Building (RB) and observed the leak to be from a valve packing and would require unit shutdown to repair. A safety evaluation pursuant to Oconee Technical Specification 3.1.6.5 was performed. At 2200 on April 8, 1978, the leak was determined to be coming from Spray Control Outlet Block Valve, 2RC-3. This valve and an instrument line root valve were both repacked. On April 12, 1978, the unit was started up but prior to achieving criticality, an additional leak was discovered in the pressurizer heater bundle. This leak was repaired on April 22, 1978. The unit was then returned to service.

Cause of Occurrence:

The leaks on 2RC-3 and the instrument line root valve were both caused by blown packing. The leak in the pressurizer heater bundle was caused by a flange seal weld failure which was evidently initiated during startup after the valve repairs.

Analysis of Occurrence:

The leakage rate experience throughout the incident was well within the capacity of one HPI pump and no adverse effect on Reactor Coolant System capabilities resulted. The leakage was entirely contained within the Reactor Building. Public health and safety were not endangered by this incident.

A total of 40.885 man-rems of exposure were received by 104 persons involved in the investigation and repair operations.

Corrective Action:

The two leaking values were repacked and the pressurizer heater bundle flange was cleaned and seal welded.

NRC FORM 366 (7.77)

U. S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT EXHIBIT A CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) j() CONT REPORT L 6 0 5 0 0 0 2 7 0 7 0 4 0 7 7 8 8 0 5 0 5 7 8 9 SOURCE 50 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) [0] 2] [On April 6, 1978, the RC leak rate exceeded 1 gpm. It was determined that the [0]] Lleak was primarily a packing leak on 2RC-3, the Spray Control Block Valve, [0]4] | the valve and an instrument line root valve were repacked. During startup, [0]5] | another leak was discovered in the pressurizer heater bundle. The weld was of repaired and the unit returned to service on April 22, 1978. No adverse [0]7 [effects resulted since all leakage was contained and no loss of RCS capability **OB** was experienced. SYSTEM CODE CAUSE CAUSE COMP VALVE COMPONENT CODE SUBCODE SUBCOOR 0 9 E X (14 18 LER/RO EVENT YEAR OCCURRENCE REVISION. REPORT NO. CODE 010 6 0 3 10 13 80 FORM SUB. [A]() SUPPLIER ⁰]@ R | 3 | 4 | CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) 10 The valve leaks were a result of blown packing. The pressurizer heater in bundle leak was caused by a flange seal weld failure evidently initiated during startup after the valve repairs. The valves were repacked and the 177 I flange was re-welded to repair the leaks. 113 14 OTHER STATUS METHOD OF DISCOVERY E 3 1003 Operator Observation 32 NA AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE (36) NA ERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 0 4 3 E38 Total exposure for investigation and repair-40.885 man-rem 1.7 ERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 0 1 8 NA 11 LOSS OF OR DAMAGE TO FACILITY (43) 80 Z (42) 1 9 NA 10 PUBLICITY DESCRIPTION (45) Y 44 Explanation of outage NRC USE ONLY 2 0 K. R. Wilson (704) 373-8197 NAME OF PREPARER PHONE: