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DUKE POWER COMPANY

Power Building 422 South Church Street, Charlotte, N. C. 28242

WILLIAM O. PARKER, JR. Vice President Steam Production March 17, 1977

REGULATORY DOCKET FILE COPY.

Mr. Norman C. Moseley, Director U. S. Nuclear Regulatory Commission Suite 818 230 Peachtree Street, Northwest Atlanta, Georgia 30303

Re: Oconee Unit 2 Docket No. 50-270

Dear Mr. Moseley:

HULL 9112

TELEPHONE: AREA 704

373-4083

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-270/77-3.

Very truly yours, William O. Parker, Jr

LJB:ge Attachment

cc: Director, Office of Management Information and Program Control DUKE POWER COMPANY OCONEE UNIT 2

Report No.: RO-270/77-3

Report Date: March 17, 1977

Occurrence Date: February 15, 1977

Facility: Oconee Unit 2, Seneca, South Carolina

Identification of Occurrence: Reactor Building pressure transmitter out of calibration

Conditions Prior to Occurrence: Unit at 100 percent full power

Description of Occurrence:

On February 15, 1977, while performing a comparison of readings on the three Reactor Building narrow range pressure transmitters, a difference of 0.6 psi was noted. Investigation revealed that Reactor Building narrow range pressure transmitter, Channel 2 2BS4-PT2, was set less conservative than the setpoint specified in Oconee Technical Specification 2.3. The pressure transmitter was promptly recalibrated to the required specifications.

Apparent Cause of Occurrence:

The cause of this incident has not been determined. Examinations of the Reactor Building pressure transmitters were made periodically over a one week span of time to determine if the transmitters were experiencing drift but no indications of setpoint drift were found.

Analysis of Occurrence:

The Reactor Building pressure is sensed by three redundant pressure transmitters which provide inputs to the Engineered Safeguards (ES) system. This incident resulted in one of the three channels being set slightly less conservative at 5.6 psi rather than at 4 psi as required by Oconee Technical Specification 2.3. The two redundant channels of the Reactor Building pressure transmitters assure that the required inputs to the ES system would have been supplied. It is therefore considered that the health and safety of the public were not affected by this occurrence.

Corrective Action:

The Channel 2 Reactor Building pressure transmitter was recalibrated to the required specifications. A check of all three pressure transmitters was made and the Reactor Protective System setpoints were verified to be within Technical Specification limits.

Examinations of the three Reactor Building pressure transmitters were periodically made over a one week time span to determine if the transmitters were experiencing any drift. No evidence of setpoint drift was discovered. No further corrective action is considered necessary.