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CONTROL NO: 8312
FILE: INCIDENT REPORT FILE

FROM: Duke Power Co. Charlotte, N. C. William O. Parker, Jr.			DATE OF DOC 8-1-75	DATE REC'D 8-5-75	LTR XXX	TWX	RPT	OTHER
TO: Norman C. Moseley			ORIG NONE	CC	OTHER	SENT AEC PDR <u>XXXX</u> SENT LOCAL PDR <u>XXXX</u>		
CLASS	UNCLASS XXXXX	PROP INFO	INPUT	NO CYS REC'D 1		DOCKET NO: 50-270		

DESCRIPTION:
Ltr. trans the following....

ENCLOSURES:

Unusual Event # 75-9, on 6-19-75, concerning
Excessive Reactor Building Emergency Hatch
Leak Rate.....

PLANT NAME: Oconee # 2

**ACKNOWLEDGE
DO NOT REMOVE**

FOR ACTION/INFORMATION

8-7-75

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AB

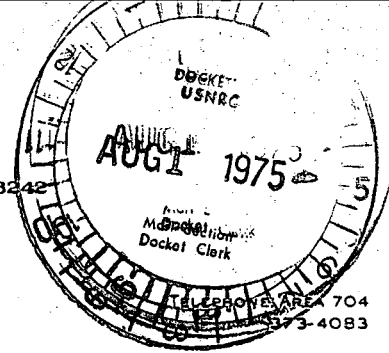
DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

August 1, 1975



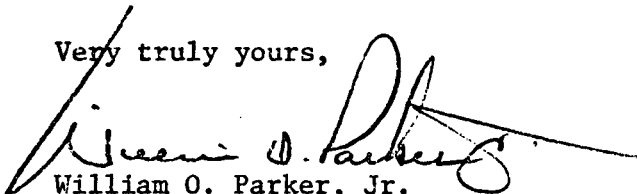
Mr. Norman C. Moseley, Director
U. S. Nuclear Regulatory Commission
Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Re: Oconee Unit 2
Docket No. 50-270

Dear Mr. Moseley:

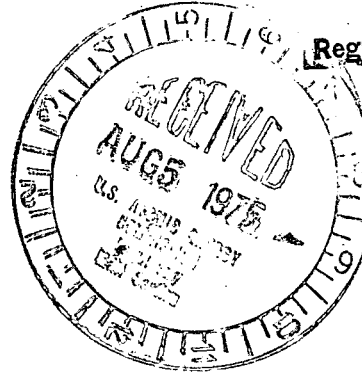
Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station
Technical Specifications, please find attached Unusual Event Report
UE-270/75-9.

Very truly yours,


William O. Parker, Jr.

ROS:ge
Attachment

cc: Mr. Angelo Giambusso



File 07a

8312

DUKE POWER COMPANY
OCONEE UNIT 2

Report No.: UE-270/75-9

Report Date: August 1, 1975

Event Date: June 19, 1975

Facility: Oconee Unit 2, Seneca, South Carolina

Identification of Event: Excessive Reactor Building Emergency Hatch Leak Rate

Conditions Prior to Event: Unit at full power

Description of Event:

On June 19, 1975, during the performance of the periodic Emergency Hatch Leak Rate Test, it was determined that the emergency hatch leakage did not meet the total containment leak rate as specified in Technical Specification 4.4.1.2.3. An investigation revealed that a cracked union fitting on an equalization line inside the emergency hatch was responsible for the leakage.

Designation of Apparent Cause of Event:

An elbow and a union fitting on the hatch equalization line were found to be contributing to the leakage. These fittings were tightened in an effort to reduce the leakage. It was later determined that the union fitting was cracked. It could not be determined when the union fitting had been cracked. However, recent modification to the interlock mechanism had required disconnection of the union fitting. The union could have been cracked when reconnected at that time or later, while being tightened subsequent to this incident.

Analysis of Event:

This periodic test is performed to assure that Reactor Building leakage would be within acceptable limits in the event of an accident. During this test, the volume between the two emergency hatch doors is pressurized to detect leakage. In this incident leakage was determined to be through a union fitting on the equalization line on the inner door. After the union fitting was replaced, the emergency hatch leakage was well below the acceptance criteria. Thus only the inner door was leaking and the outer door was capable of maintaining containment integrity. The health and safety of the public was not affected by this incident.

Corrective Action:

The cracked union fitting was replaced and the emergency hatch was retested and found acceptable.