

## **NRR-PMDAPem Resource**

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**From:** Wiebe, Joel  
**Sent:** Monday, November 30, 2015 10:43 AM  
**To:** Jessica Krejcie  
**Cc:** Joseph Bauer  
**Subject:** Followup RAI Regarding Westinghouse InfoGram (IG) 14-1 Relating to Braidwood UHS Amendment

Jessica,

Let me know if you would like a clarification call regarding this RAI. Also let me know if there is any proprietary information in this RAI.

Joel

### SCVB-RAI-15

Exelon response to SCVB-RAI-11 in the above reference letter did not provide Braidwood Units 1 and 2 plant specific information regarding the impact of the corrections resulting from Westinghouse InfoGram (IG) 14-1 on the loss-of-coolant accident (LOCA) mass and energy (M&E) release, containment pressure and temperature response, sump net positive suction head (NPSH) analysis, and minimum containment pressure for emergency core cooling system (ECCS) analysis. Exelon response refers to analysis performed for a generic large containment plant which only addresses the impact on the containment peak pressure and temperature. The response does not provide a comparison of the related parameters of the generic plant with the Braidwood plant.

Without additional quantitative information, the NRC staff evaluation of Braidwood Units 1 and 2 containment performance due to increase in ultimate heat sink (UHS) temperature cannot be completed. After correcting the RCS metal specific heat and density to ASME values reported in IG 14-1, the NRC staff requests the following:

- (a) Revise the analyses of record for the above containment analyses, and provide results. As an alternative, by performing Braidwood Units 1 and 2 plant specific sensitivity analysis, provide increase in containment peak pressure, peak temperature, sump water temperature and decrease in the available NPSH for the pumps that draw suction flow from the sump during the LOCA recirculation phase..
- (b) Revised Technical Specification (TS) values of Integrated Leak Rate Test (ILRT) pressure ' $P_a$ '. In case  $P_a$  increases from its current TS value, provide an evaluation of the impact on the current ILRT and Local Leak Rate Test (LLRT) results.
- (c) Impact on the residual heat removal (RHR) heat exchanger heat load and on the discharge temperature of the closed cooling water (CC) from the RHR heat exchanger.
- (d) Impact on the reactor containment fan cooler (RCFC) heat load and on the exit temperature of the essential service water (SX) from the RCFC.
- (e) Overall impact on the temperature of SX exiting the CC heat exchanger.
- (f) Overall impact on the temperature of SX returning to the UHS.

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