

NRC INSPECTION MANUAL

IRIB

INSPECTION PROCEDURE 71153

FOLLOW-UP OF EVENTS AND NOTICES OF ENFORCEMENT DISCRETION

PROGRAM APPLICABILITY: 2201 Appendix A, 2515 Appendix A

EFFECTIVE DATE: 01/01/2016

CORNERSTONES: ALL

INSPECTION BASIS: In accordance with Management Directive (MD) 8.3, "NRC Incident Investigation Program," U.S. Nuclear Regulatory Commission (NRC) management makes decisions regarding the level of investigatory response for a significant operational event or degraded condition. These decisions are based upon deterministic and risk criteria. On-site inspectors review plant status and equipment/operator performance in order to provide inputs to NRC management and risk analysts as required to evaluate event/degraded condition regarding the deterministic and risk criteria.

LEVEL OF EFFORT: Inspector effort may vary significantly based on the complexity and number of operational events, degraded conditions, licensee event reports (LERs), personnel performance issues, and Notices of Enforcement Discretion (NOEDs) reviewed.

71153-01 INSPECTION OBJECTIVE

01.01 Evaluate licensee events and degraded conditions for plant status and mitigating actions in order to provide input in determining the need for an Incident Investigation Team (IIT), Augmented Inspection Team (AIT), or Special Inspection (SI).

01.02 Review written LERs.

01.03 Review personnel performance during planned non-routine plant evolutions and/or contribution to unplanned non-routine evolutions, events and transient operations.

01.04 Review granted NOEDs.

71153-02 INSPECTION REQUIREMENTS

02.01 Event Follow-Up

- a. Collect the information necessary to communicate the event details to supervision and management for a determination of the appropriate agency response. Observe plant parameters and status.
- b. Evaluate performance of mitigating systems and licensee actions.
- c. Confirm that the licensee properly classified the event in accordance with emergency action level procedures and made timely notifications to NRC and state/county governments, as required (**Title 10 of the Code of Federal Regulations (10 CFR) Parts 20, 50.9, 50.72**).
- d. Communicate details regarding the event to management, risk analysts and others in the Region and Headquarters as input to determining the need for an IIT, AIT, or SI.
- e. Retain observations related to apparent performance issues and contributing factors for potential follow-up by the IIT, AIT, SI, or appropriate Reactor Oversight Process (ROP) baseline inspection.

02.02 LERs

- a. Review LERs and related documents regarding the accuracy of the LER (e.g., based on independent NRC observations in an SI), appropriateness of corrective actions, violations of requirements, and generic issues (including the applicability of Part 21 reporting requirements).
- b. Optionally within the allotted resources, review Event Notification retractions to ensure they are in accordance with 10 CFR 50.72, "Immediate Notification Requirements for Operating Nuclear Power Reactors."

02.03 Personnel Performance

- a. Evaluate operator performance in planned and unplanned non-routine events and transients. Independently evaluate the initiating cause of reactor trips involving operator errors, and the personnel response to reactor trips requiring more than routine operator actions. Determine if the personnel response was appropriate and in accordance with procedures and training.
- b. For unplanned occurrences, where personnel error was an initiating cause, review operator logs, plant computer data, or strip charts after stable plant operations have been resumed. Determine if the personnel response was appropriate.

- c. In the off normal or transient condition during planned non-routine evolutions, review the plan for the evolution, procedures, briefings, and contingency plans. Determine if the personnel response was appropriate.
- d. In all cases, review operator logs, computer data, recorder data, procedural requirements, and related training to aid in the assessment of personnel response. Determine if the personnel response was appropriate.

02.04 NOEDs. Review NOEDs and related documents to verify the accuracy of the NOEDs and their consistency with licensee oral assertions, and implementation of compensatory measures and commitments. In addition, assess the adequacy of licensee corrective actions and root cause determinations, and determine if the NRC requirements have been violated.

71153-03 INSPECTION GUIDANCE

MD 8.3, "NRC Incident Investigation Program," defines a significant operational event as radiological, safeguards, or other safety-related operational event at a NRC-licensed facility that poses an actual or potential hazard to public health and safety, property, or the environment. At power reactors, these events include significant unplanned degraded conditions identified by the licensee or NRC.

The staff uses MD 8.3 and Inspection Manual Chapter (IMC) 0309, "Reactive Inspection Decision Basis for Reactors", to determine the appropriate NRC response to a significant operational event.

Following notification of an event or during an event, the responding on-site inspectors provide details regarding plant status and performance of equipment and personnel to management, event review staff and regional and headquarters risk analysts. The details are used to determine the level of agency response, investigatory response if any, i.e., IIT, AIT, or SI, and any special resources and expertise needed for event follow-up. If no reactive inspection is warranted in accordance with MD 8.3 and IMC 0309, the event would be followed up through the applicable ROP baseline inspection procedure(s).

Appendix A illustrates the relationship between event response and the ROP.

Appendix B provides guidance for limiting NRC's impact on licensees during an event.

Appendix C provides guidance for plant response and event follow-up.

Specific Guidance

03.01 Event Follow Up

- a. Obtain understanding of plant status, equipment/personnel performance and plant management decisions to assist NRC management in making an informed evaluation of

plant conditions. Observe plant parameters and status for mitigating systems/trains and fission product barriers. Information sources include drawings, system descriptions, control board indications, plant logs, computer data, recorders, and licensee personnel. Refer to Appendix C for additional specific guidance for Pressurized Water Reactors (PWRs), Boiling Water Reactors (BWRs), and Post Transient Response.

- b. Evaluate whether the licensee has appropriately resolved event issues prior to restart, where applicable, such as by attending meetings of the Plant Oversight Review Committee.
- c. No specific guidance.
- d. MD 8.3 and IMC 0309 provide deterministic criteria which are applicable to power reactors. Inspectors provide details which help determine whether the event meets the deterministic criteria. An IIT, AIT, or SI is considered for certain events or degraded conditions meeting deterministic criteria without any probabilistic risk input, e.g., exceed a safety limit of the licensee's technical specifications, site area emergency, significant radiological release, significant occupational or public exposure, and safeguards concerns.

Other deterministic criteria related to events or degraded conditions are risk informed, e.g., loss of a safety function or multiple failures in systems used to mitigate an event. For events meeting these criteria, risk analysts estimate Conditional Core Damage Probability (CCDP). For degraded conditions meeting these criteria, risk analysts estimate Incremental Conditional Core Damage Probability (ICCDP). These estimates are often based only on best available information at an early stage in the development of the facts. If a quantitative CCDP cannot be obtained, the risk analyst provides qualitative risk insights. In all cases, the risk guidelines of MD 8.3 and IMC 0309 are not prescriptive, and are to be used with an understanding of the greatest uncertainties. When appropriate, risk analysts can also evaluate containment performance by calculating Conditional Large Early Release Probability (CLERP) or Incremental Conditional Large Early Release Probability (ICLERP).

The above process is described in IMC 0309 and MD 8.3, Part I, Pages 4 through 8. They include tables which recommend appropriate event response options (IIT, AIT, or SI) as a function of CCDP (or ICCDP) and CLERP (or ICLERP).

To assist risk analysts, inspectors provide input (in addition to a and b above), such as equipment malfunctions/unavailabilities and operator errors. Inspectors verify the availability of mitigation equipment not required to operate during the event, but which could contribute to increased risk if unavailable. If the event corresponds to a significance determination process (SDP) Phase 2 worksheet (e.g., transient, loss of offsite power), the worksheets can identify the most likely core damage sequences that include known failure of equipment and/or operator error and the remaining mitigation capability for reactor safety. The inspector should verify the availability of this mitigation capability.

- e. *Inspectors should provide the follow-up inspection team leader with any information on potential contributing factors that may assist the follow up assessment of the event. Information should include any issues noted with components of safety culture as described in IMC 0310, "Components within the Cross-cutting Areas." Information about observing a safety conscious work environment is contained in IP 71152, "Problem Identifications and Resolution." The information is provided for follow-up by IIT, AIT, SI, or ROP baseline inspection(s). The staff assigned to review the event as the agency response are responsible for documentation in accordance with the procedure governing the activity. [C1]*

03.02 LERs. Review written LERs, but not telephone notifications to the NRC Operations Center for invalid actuations, as allowed in 10 CFR 50.73. LERs that involve operator errors are reviewed under Section 02.03. Licensee resolution of issues may be addressed under the Problem Identification and Resolution sections of individual baseline inspection procedures. IMC 0612, "Power Reactor Inspection Reports," covers documentation of LER reviews.

03.03 Personnel Performance. This inspectable area is intended to review personnel performance which contributed to off-normal and transient conditions, as well as operator response to these conditions. In most cases, since these occurrences are unplanned, the inspector will not directly observe operator performance and will be required to review the occurrence and operator response after stable plant operations have been resumed.

- a. Independently evaluate the initiating cause of any reactor trip involving operator errors, and evaluate the personnel response to any reactor trip which involved more than routine expected operator actions in the response of the trip. Determine if the response was appropriate to the event and in accordance with procedures and training.
- b. If the off normal or transient condition occurred during a planned non-routine evolutions, review the plan used for the evolution, procedures, briefings, contingency plans, and determine how personnel performed during the evolution.
- c. In all cases, review operator logs, computer data, recorder data, procedural requirements, and related training to aid in the assessment of personnel response.
- d. Determine if the licensee's evaluation appropriately assessed operator performance.

03.04 NOEDs. **IMC 0410** "Notices of Enforcement Discretion," Regulatory Issue Summary 2005-001, "Changes to Notice of Enforcement Discretion (NOED) Process and Staff Guidance," NRC Enforcement Manual and Policy covers NRC NEOD regarding conditions such as limiting conditions for operation in power reactor technical specification. Refer to the Office of Enforcement's (OE's) website (<http://www.nrc.gov/reading-rm/doc-collections/enforcement/notices/more.html>) for additional information on enforcement and NOEDs. OE approval is required to exercise enforcement discretion (i.e., to disregard taking enforcement actions such as a notice of violation or non-cited violation) when a more than a minor violation exists.

71153-04 RESOURCE ESTIMATE

Annual hours for one unit: 65; two units: 80; three units: 100. Annual hours are for planning purposes only. Hours may vary depending on the complexity and number of operational events, degraded conditions, LERs, personnel performance issues, and NOEDs that have occurred for a given site.

71153-05 COMPLETION STATUS

Inspection of the minimum sample size constitutes completion of this procedure in the Reactor Programs System inspection tracking system. Minimum sample size (regardless of the number of units) is 1 for event follow-up and NOEDs. Event follow-up includes non-routine and transient operations involving personnel performance, all reactor trips requiring more than routine operator response, and all LERs.

71153-06 REFERENCES

[10 CFR 50.72](#), "Immediate notification requirements for operating nuclear power reactors."

[10 CFR 50.73](#), "Licensee event report system."

[NRC Enforcement Manual](#)

[NRC Enforcement Policy](#)

[NUREG-1022](#), "Event Reporting Guidelines: 10 CFR 50.72 and 50.73"

[Management Directive 8.3](#), "NRC Incident Investigation Program"

[Inspection Manual Chapter 0310](#), "Aspects within the Cross-cutting Areas"

[Inspection Manual Chapter 0309](#), "Reactive Inspection Decision Basis for Reactors"

[Inspection Manual Chapter 0410](#), "Notices of Enforcement Discretion"

[Inspection Manual Chapter 0609](#), "Significance Determination Process"

[Inspection Manual Chapter 0612](#), "Power Reactor Inspection Reports"

[Inspection Procedure 71152](#), "Problem Identification and Resolution"

[Inspection Procedure 93800](#), "Augmented Inspection Team"

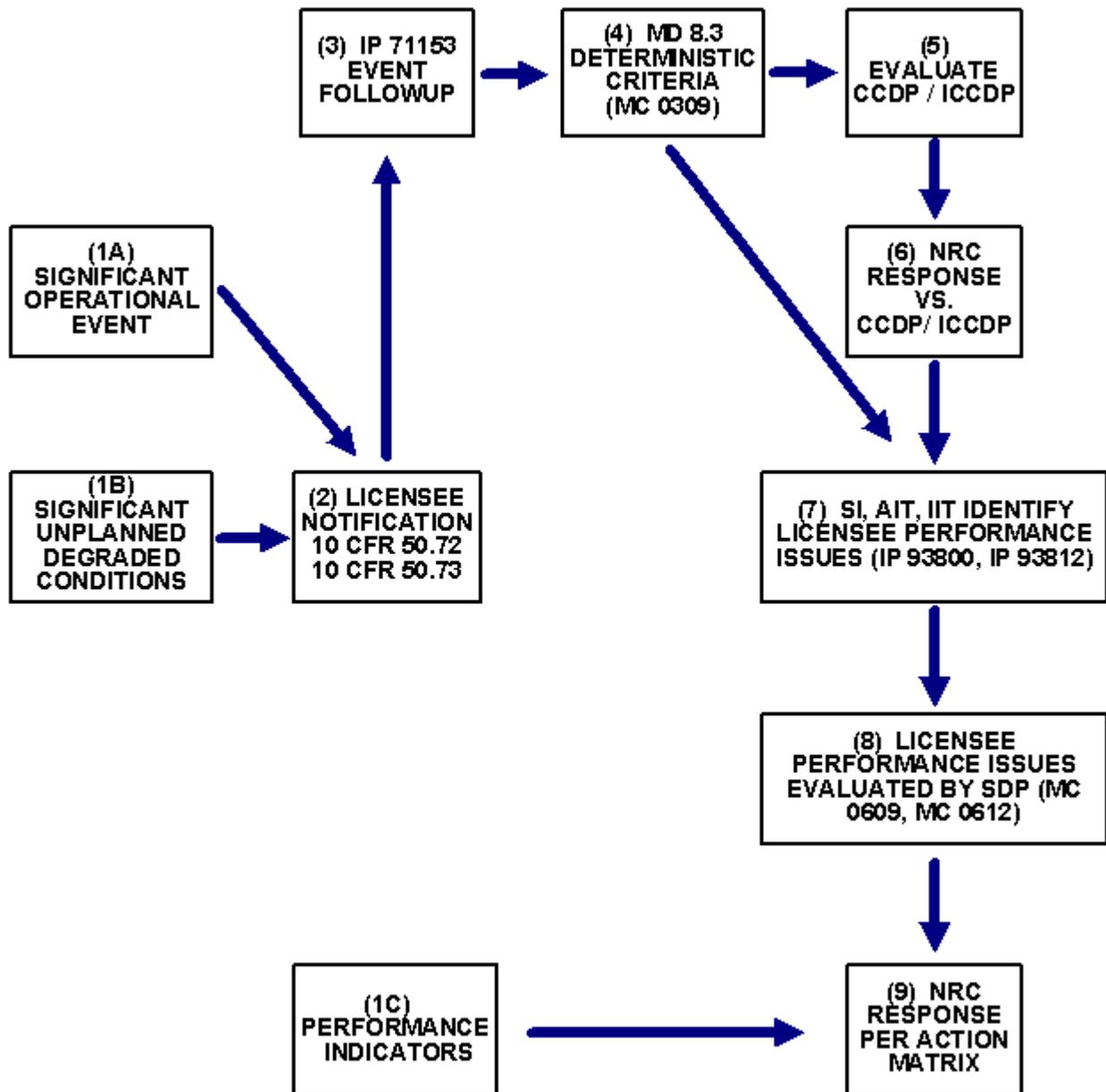
[Inspection Procedure 93812](#), "Special Inspection"

[Regulatory Issue Summary 2005-001](#), “Changes to Notice of Enforcement Discretion (NOED) Process and Staff Guidance”

END

Appendix A

RELATIONSHIP OF EVENT FOLLOW-UP TO REACTOR OVERSIGHT PROCESS



BLOCK 1A — A significant operational event is a radiological, safeguards, or other safety-related operational event that poses an actual or potential hazard to public health and safety, property, or the environment.

BLOCK 1B - Significant unplanned degraded conditions may be identified by the licensee or NRC. Plant configurations due solely to planned maintenance need not be considered.

BLOCK 1C — Performance indicator (PI) thresholds are in units of change in annualized Core Damage Frequency (delta annualized CDF). Some events, such as reactor trips may also be counted in PI data.

BLOCK 2 — Licensee notification occurs.

BLOCK 3 — Licensee notifications in accordance with 10 CFR 50.72 are one means of activating IP 71153 initial event follow-up by on-site NRC inspectors. If an on-site inspector is not immediately available, this responsibility transfers to the Headquarters Operations Officer until regional personnel can respond.

BLOCK 4 — MD 8.3, Part I and IMC 0309 include deterministic criteria. Events meeting criteria which are not risk informed may result in IITs, AITs, or SIs. Power reactor events/degraded conditions meeting risk informed criteria are evaluated for CCDP/ICCDP or CLERP/ICLERP.

BLOCK 5 — For events, risk analysts use NRC's Standardized Plant Analysis of Risk models and other available tools to estimate CCDP, which accounts for all equipment unavailability, regardless of cause. For degraded conditions, ICCDP is used for risk significance. Initial estimates of CCDP/ICCDP may be made within 4-8 hours of receiving relevant information. Inspectors support risk analysts by providing event details such as equipment malfunction/unavailability, operator errors, and equipment out of service for maintenance. For assessing degraded condition exposure time, assume T/2 if time of unavailability is unknown. T is the time interval from the last known operability until discovery of the degraded condition. Inspectors verify availability of mitigation equipment or containment function that were not required during the event, but which could contribute to increased risk if unavailable. Inspectors use plant-specific SDP phase 2 worksheets to gain qualitative risk insights of events.

BLOCK 6 — The table in IMC 0309 and on Page 6 of Management Directive 8.3, Part I, lists appropriate power reactor operational event response options (IIT, AIT, SI) as a function of CCDP (or ICCDP). This determination considers the uncertainty of influential assumptions and their effect on risk significance.

BLOCK 7 — SIs, AITs, and IITs evaluate events/degraded conditions and their root causes, and identify licensee performance issues.

BLOCK 8 — Licensee performance issues are evaluated with the SDP (considering only performance-related equipment unavailabilities), placing the issues in delta annualized CDF bands.

BLOCK 9 — Because PI thresholds are in units of delta annualized CDF, PIs and SDP results are combined in the NRC Action Matrix to determine agency responses to the performance issues identified by the event response.

END

Appendix B

LIMITING NRC IMPACT DURING EVENTS

I. Inspector Conduct While in the Control Room

For plant events, inspectors must perform sufficient inspection to develop an independent assessment of plant conditions, which will be used in making decisions on NRC's responses to an event. Activities that form the basis for this assessment may include independent measurements, verifying the accuracy of information, control board walkdowns (to observe annunciators, process parameters, switch positions, and other instrumentation), or assessment of licensed operator performance during ongoing activities.

The NRC's goal is to monitor and assess with as little impact on the licensee as possible and at the same time ensure NRC evaluations are timely and accurate. During plant events, timely and independent inspector assessments are crucial; however, the degree of interaction with operators may be limited in light of ongoing control room activities. The inspector must use judgement in establishing a balance between obtaining necessary information and not being intrusive in licensee response activities. The appropriate balance involves numerous variables, including safety significance of the event, complexity of the event, time constraints, and available staff.

The following guidance is provided to establish consistency for inspector conduct in the control room. When the NRC activates its emergency response plan, inspectors should follow the guidance in the applicable emergency response procedure. This guidance is intended for use in situations where the NRC has not activated its emergency response plan; however an abnormal event has happened at the plant. Inspectors should note that some of the guidance, such as inspector location in the control room and not interrupting operators, apply to all emergency situations. While this guidance deals mainly with event responses, specific attributes are applicable to inspector interaction with operators during normal conditions both in and outside the control room.

- a. During the initial response to events, the assigned senior resident inspector (SRI) or the inspector acting in this capacity is in charge of all other NRC inspectors. These inspectors will take their direction from the SRI.
- b. The number of inspectors in the control room at any given time should be the minimum number needed to accomplish the agency's work. Typically there should be only one inspector in the control room during an event, unless special circumstances warrant additional inspectors. If several inspectors or other NRC personnel are in the control room during an event, the SRI or resident inspector will be in charge of them and will determine and communicate to the other inspectors and personnel what, if any, assistance is needed.
- c. Inspectors will adhere to the licensee's established administrative policies regarding entry into the restricted or "at the controls" area of the control room. For example, the inspector may need to ask the control room SRO or RO for permission to enter the

restricted area. Under no circumstances should the inspector demand entry into the restricted area. If such entry is denied, the inspector should escalate the request to the licensee's management and inform NRC management of the problem. For general access to the control room, the licensee's policy should recognize that inspector access will be unannounced. Inspectors who do not routinely enter the control room should identify themselves to the operators when they enter the control room.

- d. While observing ongoing activities in the control room, the inspector should be in a location which is out of the way of operators and does not obstruct their view of the reactor controls and instrumentation, yet the location provides the inspector with a broad view of the control room. An acceptable location outside the restricted "at the controls" area is preferable. It is recognized that short amounts of time in the restricted area may be necessary at appropriate stable time periods to verify significant parameters.
- e. Operators should not be interrupted, questioned or otherwise distracted from performing their duties while responding to an event or while performing other duties where their attention must be focused on the task at hand. Also, inspectors should not interfere, interrupt, or otherwise disturb communications between operators and communications between operators and their supervision.
- f. If an inspector identifies a significant problem or question about plant or operator safety that needs to be addressed in an urgent manner, then the inspector should discuss it quickly and quietly at a time when it will not interrupt ongoing operator actions. This discussion should be held with the shift supervisor or emergency response manager. However, it may be appropriate to interrupt the operator if the inspector feels that an operator action may endanger plant personnel or the plant. Inspectors should hold their non-urgent questions for a more appropriate time.
- g. NRC personnel communicating with off-site organizations should generally do so from outside of the control room. Communication is possible from the NRC phone in the TSC or other phones outside the control room that have been agreed to with the licensee. It is acceptable for the inspector to make a phone call from the control room provided the licensee agrees to the use of the phone and the phone conversation will not disrupt control room activities.
- h. Because of the authoritative role of the NRC, licensees listen carefully to inspectors and may interpret statements, side remarks, or observations as directives or requirements. Consequently, open, clear, and direct communications between inspectors and licensees are particularly important during events.

II. Conference Calls With Licensees During an Ongoing Event

When initially responding to an event, the NRC is dependent upon information provided by licensees and inspectors at the plant (typically resident inspectors). This information is used for initially assessing events and making decisions about how to respond to the event. The NRC

typically gets this initial information from licensees through their notification to the NRC Operations Center pursuant to 10 CFR 50.72 or from conference calls between the NRC staff and the licensee. The NRC values conference calls as an efficient method of obtaining accurate and timely information. Such calls promote a mutual understanding of the facts and any concerns.

Caution is needed in scheduling and conducting conference calls when the calls are held during an ongoing event or situations where heightened licensee attention is being directed to a plant evolution. While information obtained in a conference call is extremely valuable to the NRC's overall understanding of a plant event, the overriding goal is that the call will not interfere or detract from the licensee's ability to safely operate the plant. The following guidance should be used for conducting conference calls with licenses during abnormal plant conditions. Examples of abnormal plant conditions would be the declaration of a Notification of Unusual Event (NOUE) or the use of an emergency operating procedure (EOP).

- a. NRC management should decide whether a conference call with the licensee is needed and if conducting a conference call is appropriate at that particular time. NRC management may want to discuss with senior licensee management the possibility of conducting a conference call. The stability of the plant is the primary factor in deciding on a conference call. Other factors to be considered in this decision include: the current level of NRC staff understanding and information available for the event; the safety significance of the event; the complexity of the event; and the current level of licensee activity in mitigating the event.
- b. Generally the licensee should be informed of the NRC's desire to have a conference call by the senior resident inspector or resident inspector if they are available. The licensee must be included in deciding the most appropriate time for the call so that the call does not interfere with plant response activities. Also the licensee should decide which individuals from their staff will participate in the call.

When requesting the conference call, the licensee must be clearly informed of the NRC's desire that the conference call not interfere with their response to plant conditions and that delaying the call is a valid option for them.

- c. NRC technical staff and management with the right background should participate in the conference call to ensure proper questioning and understanding of the event and associated issues. The senior NRC manager on the call should identify his/her self and is responsible for ensuring that the conference call discussions are properly focused on important issues and that side issues are discussed at another time.
- d. If time allows, an agenda for the conference call should be developed to ensure the call remains properly focused. The licensee should be informed of the proposed discussion topics and planned NRC participants to allow the licensee to prepare for the call.
- e. Any follow-up actions resulting from the conference call should be summarized at the end of the call by an NRC manager to ensure the licensee clearly understands and agrees with the actions.

END

Appendix C

PLANT RESPONSE AND EVENT FOLLOW-UP

- I. Basis for Actions Specified in Each Checklist Section
 - a. Immediate Actions
 - Intended to provide NRC management sufficient information to assess plant stability and make decisions regarding initial agency response without overwhelming the on-site inspector with information requests.
 - Based on conditions monitored by operators when implementing emergency operating procedures.
 - In most cases, can be easily collected over the phone by an off-site inspector or collected and independently verified by an on-site inspector in less than 15 minutes.
 - b. Short Term Considerations
 - Intended to provide NRC management sufficient information to identify potential challenges to plant stability; verify the integrity of the safety barriers and assess radiological impacts.
 - Prompts management and inspector decisions related to communication and coordination.
 - c. Event Follow-up
 - Provides a list of post-trip review activities that inspectors are expected to complete when implementing Inspection Procedure 71153, Event Follow-up.

Pressurized Water Reactor Plant Transient Response

For each item gather information from the licensee or through direct observation regarding plant conditions (i.e. Rx power - "shutdown,"), control method ("all but three rods inserted"), assessment (i.e. Abnormal), and if the assessment is abnormal what actions are being taken and what procedure is being used?

<u>Immediate Actions</u>	<u>Event Time:</u>	<u>Event Date:</u>
<u>Cause/Description of Transient:</u>		
<input type="checkbox"/> Reactor Power		
<input type="checkbox"/> Turbine Status		
<input type="checkbox"/> RCS pressure		
<input type="checkbox"/> Electrical Power		
<input type="checkbox"/> ECCS Actuation		
<input type="checkbox"/> Decay Heat Removal Path		
<input type="checkbox"/> Steam Generator Pressure and Level		
<input type="checkbox"/> EOPs (entered or imminent)		
<input type="checkbox"/> EALs (declared or imminent, time entered/exited)		
<input type="checkbox"/> Radioactive release (none, occurred & terminated, on-going or imminent?)		
<input type="checkbox"/> Effect on other unit(s)		
<input type="checkbox"/> Security event (threat, damage)		
<u>Short Term Considerations</u>		
<input type="checkbox"/> Fuel Clad, Reactor Coolant System, and Containment Barriers		
<input type="checkbox"/> Tech Spec Safety Limits		
<input type="checkbox"/> EAL progression path		
<input type="checkbox"/> Safety Related Equipment out of service or failed to actuate		
<input type="checkbox"/> Support systems for Safety Related Equipment functioning		
<input type="checkbox"/> What are the Licensee's priorities		
<input type="checkbox"/> Abnormal RCS or Secondary temperatures or pressures		
<input type="checkbox"/> Steam Generator and Pressurizer PORV's and Safeties (actuated during event?)		
<input type="checkbox"/> Operator Response in accordance with procedures		
<input type="checkbox"/> Licensee notifications to the state, locals, and NRC		
<input type="checkbox"/> Communicate safety concerns to NRC and Licensee management		
<input type="checkbox"/> Personnel injuries, contaminations, or overexposures		
<input type="checkbox"/> Additional support or relief needed to continue to monitor the event		

Boiling Water Reactor Plant Transient Response

For each item gather information from the licensee or through direct observation regarding plant conditions (i.e. Rx power - “shutdown,”), control method (“all but three rods inserted”), assessment (i.e. Abnormal), and if the assessment is abnormal what actions are being taken and what procedure is being used?

<u>Immediate Actions</u>	<u>Event Time:</u>	<u>Event Date:</u>
<u>Cause/Description of Transient:</u>		
<input type="checkbox"/> Reactor Power		
<input type="checkbox"/> RPV pressure		
<input type="checkbox"/> RPV level		
<input type="checkbox"/> Electrical Power		
<input type="checkbox"/> ECCS Actuation		
<input type="checkbox"/> Decay Heat Removal Path		
<input type="checkbox"/> Primary Containment		
<input type="checkbox"/> Suppression Pool		
<input type="checkbox"/> SRV's (closed, cycling or stuck open?)		
<input type="checkbox"/> EOPs (entered or imminent)		
<input type="checkbox"/> EALs (declared or imminent, time entered/exited)		
<input type="checkbox"/> Radioactive release (none, occurred & terminated, on-going or imminent?)		
<input type="checkbox"/> Effect on other unit(s)		
<input type="checkbox"/> Security event (threat, damage)		
<u>Short Term Considerations</u>		
<input type="checkbox"/> Fuel Clad, Reactor Coolant System, and Primary and Secondary Containment		
<input type="checkbox"/> Tech Spec Safety Limits		
<input type="checkbox"/> Electrical Power		
<input type="checkbox"/> EAL progression path		
<input type="checkbox"/> Safety Related Equipment out of service or failed to actuate		
<input type="checkbox"/> Support systems for Safety Related Equipment functioning		
<input type="checkbox"/> What are the Licensee's priorities		
<input type="checkbox"/> Operator Response in accordance with procedures		
<input type="checkbox"/> Licensee notifications to the state, locals, and NRC		
<input type="checkbox"/> Communicate safety concerns to NRC and Licensee management		
<input type="checkbox"/> Personnel injuries, contaminations, or overexposures		
<input type="checkbox"/> Additional support or relief needed to continue to monitor the event		

Post Transient Response

<u>Event Follow-up</u>
Provide information to the PAOs and SLOs concerning the event.
Implement IP 71153, "Follow-up of Events and Notices of Enforcement Discretion"
Confirm the licensee properly classified the event and timely notified state, locals, and NRC
Review Event Notification
Collect alarm data printouts
Obtain and review sequence of event recorders
Provide necessary information to the region in support of an IMC 0309 review and assessment.
Perform Control Room walk down, review control board status, strip charts / digital recorders.
Review Operator Logs
Walk down affected equipment
Should plant equipment be quarantined
Observe Operator interviews or review Operator statements
Determine the event chronology.
Evaluate operator response to the transient
Independently interview Operators
Evaluate the plant data of the event and compare it with the design data and FSAR descriptions to evaluate if the plant response was within the bounds of the FSAR analysis.
Review all CRs generated as a result of the transient
Evaluate the licensee's Post Trip Review for necessary Corrective Actions prior to startup.
Attend startup PORC to ensure licensee has appropriately resolved any issues identified as a result of the event
Transition to IP 71111.20, "Refueling and Other Outage Activities" as appropriate
For security related events, evaluate the communications and coordination between the security and operations organizations. Determine if a security inspector is needed for further follow up.

Wallet Cards

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Attachment 1 – Revision History for IP 71153

Commitment Tracking Number	Accession Number Issue Date Change Notice	Description of Change	Description of Training Required and Completion Date	Comment and Feedback Resolution Accession Number (Pre-Decisional, Non-Public)
N/A	04/04/00 CN 00-005	IP 71153 (Event Follow-up) is revised to provide inspection requirements and guidance for review of event reports.	None	N/A
N/A	03/06/01 CN 01-006	IP 71153 (Event Follow-up) has been revised to better define the scope of the IP, to expand the definition of power reactor events to include degraded conditions, and to integrate the IP with the options for inspection activities related to the deterministic and risk criteria in MD 8.3.	None	N/A
N/A	01/17/02 CN 02-001	IP 71153 (Event Follow-up) has been revised to delete the previous Appendix A since that material was included in Management Directive 8.3. It also clarifies that written LERs are to be reviewed, but not telephone notifications to the NRC Operations Center for invalid actuations, as allowed in 10 CFR 50.73.	None	N/A
N/A	04/16/02 CN 02-017	IP 71153 (Event Follow-up) has been revised to provide guidance on assessing degraded condition exposure time if time of unavailability is unknown.	None	N/A
N/A	06/24/03 CN 03-020	IP 71153 (Event Follow-up) this clarifies that the risk metric for events is Conditional Core Damage Probability (CCDP) and the metric for degraded conditions is incremental CCDP. Also this revision lists examples of events addressed by this IP in cornerstones outside of reactor safety.	None	N/A

Commitment Tracking Number	Accession Number Issue Date Change Notice	Description of Change	Description of Training Required and Completion Date	Comment and Feedback Resolution Accession Number (Pre-Decisional, Non-Public)
C1	ML0615605040 6/22/06 CN-06-015	Incorporate safety culture into inspection procedures. "Staff Requirements - SECY-04-0111 - Recommended Staff Actions Regarding Agency Guidance in the Areas of Safety Conscious Work Environment and Safety Culture" August 30, 2004 Revision history reviewed for the last four years.	Inspector training on use of safety culture in the ROP. 07/01/06	ML061570089
N/A	ML061920454 07/26/06 CN-06-018	Add inspections of (1) personnel performance during planned non-routine plant evolutions and/or contribution to unplanned non-routine evolutions, events and transient operations (previously in IP 71111.14); and (2) granted Notices of Enforcement Discretion.	None	ML061770161
N/A	ML080280581 02/12/08 CN 08-007	Clarified the process of using both MD 8.3 and IMC 0309 to determine appropriate responses to significant operational events.	None	N/A
N/A	ML093620322 02/02/10 CN 10-004	Added requirement to review retractions of Event Notifications, and reduced resources based on actual reported hours.	None	N/A
N/A	ML102810102 12/05/11 CN 11-039	Added guidance for inspecting personnel performance (ROPFF 71153-1554), Added Appendix C to enhance plant status data collection (ROPFF 71153-1661), updated references, adjusted resources to reflect the 2011 ROP Realignment (ML11178A329).	None	N/A
N/A	ML15322A151 12/17/15 CN15-030	Changed outdated references (ROPFF 71153-1901)	None	71153-1901 ML15349A022

Issue Date: 12/17/15
Effective Date: 01/01/16

Att1-2

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