

NRR-PMDAPem Resource

From: Purnell, Blake
Sent: Tuesday, November 17, 2015 3:07 PM
To: Loomis, Thomas R:(GenCo-Nuc) (thomas.loomis@exeloncorp.com)
Cc: Poole, Justin; James.Barstow@exeloncorp.com
Subject: Calvert Cliffs, Nine Mile Point, and R. E. Ginna - Request to Use ASME Code Case N-789 as an Alternative to ASME Code Requirements

Mr. Loomis:

By application dated October 28, 2015 (ADAMS Accession No. ML15301A596), Exelon Generation Company, LLC (the licensee) submitted a request in accordance with Paragraph 50.55a(z)(2) of Title 10 of the *Code of Federal Regulations* (10 CFR) for a proposed alternative to the requirements of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (ASME Code) for Calvert Cliffs Nuclear Power Plant, Units 1 and 2, Nine Mile Point Nuclear Station, Units 1 and 2, and R. E. Ginna Nuclear Power Plant. The proposed alternative would allow the licensee to use ASME Code Case N-789, "Alternative Requirements for PAD Reinforcement of Class 2 and 3 Moderate-Energy Carbon Steel Piping for Raw Water Service, Section XI, Division 1," in lieu of specified ASME Code requirements.

The purpose of this email is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this proposed alternative. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Pursuant to 10 CFR 50.55a(z), the applicant shall demonstrate that the proposed alternatives would provide an acceptable level of quality and safety, or that compliance with the specified requirements of 10 CFR 50.55a would result in hardship or unusual difficulty without a compensating increase in the level of quality or safety.

The NRC staff has reviewed your application and concluded that it provides technical information in sufficient detail to enable the staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed relief request in terms of regulatory requirements and the protection of public health and safety and the environment. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the staff's detailed technical review by separate correspondence.

Sincerely,

Blake Purnell, Project Manager
Plant Licensing Branch III-2 and
Planning and Analysis Branch
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission

Docket Nos. 50-317, 50-318, 50-220, 50-410, and 50-244

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"Poole, Justin" <Justin.Poole@nrc.gov>

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"Loomis, Thomas R:(GenCo-Nuc) (thomas.loomis@exeloncorp.com)" <thomas.loomis@exeloncorp.com>

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