

December 9, 2015

MEMORANDUM TO: Kevin Hsueh, Chief
Licensing Processes Branch
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

FROM: Joseph J. Holonich, Senior Project Manager */RA/*
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SUBJECT: SUMMARY OF NOVEMBER 6, 2015, CONFERENCE CALL WITH
THE ELECTRIC POWER RESEARCH INSTITUTE ON MATERIALS
RELIABILITY PROGRAM (MRP)-227-A, "PRESSURIZED WATER
REACTOR INTERNALS INSPECTION AND EVALUATION
GUIDELINES"

On November 6, 2015, the U.S. Nuclear Regulatory Commission (NRC) staff held a conference call with representatives from the Electric Power Research Institute (EPRI). The purpose of the call was to discuss clarification of MRP-227-A "Pressurized Water Reactor Internals Inspection and Evaluation Guidelines," action items (AIs) 1 and 7. Information related to the conference call can be found in the Agencywide Documents Access and Management System (ADAMS) package for the meeting at Accession No. ML15271A059.

The NRC staff began by noting there was an action from the July 15, 2014, meeting on cast austenitic stainless steel and the March 31, 2015, meeting on revisions to MRP-227-A where the NRC staff committed to provide clarification for AIs 1 and 7.

The conference call centered on the NRC staff presentation that can be found in the ADAMS package referenced previously.

During the discussion on AI 1, it was suggested that a meeting be scheduled in the spring 2016 to allow industry to review the options it sees could be used to address AI 1 and recommendations on which options might be implemented. Because the meeting would cover proprietary information, it was agreed the meeting would be noticed but closed to public participation.

A second point raised during the AI 1 portion of the NRC staff presentation was that all Babcock and Wilcox (B&W) plants had completed what was needed. The NRC staff agreed that all B&W plants were complete.

For AI 7, the NRC staff reported that the draft safety evaluation (SE) for Boiling Water Reactor Vessel Internals Project (BWRVIP)-234, "Thermal Aging and Neutron Embrittlement Evaluation of Cast Austenitic Stainless Steel for BWR Internals," would be complete and issued mid to late January 2016. BWRVIP-234 contains screening criteria that are applicable to AI 7. It was agreed that once the draft SE was issued, a meeting be held in late January to discuss how industry would address AI 7.

Actions from the meeting were:

- 1) Schedule a meeting in spring 2016 to discuss options and recommendations to address AI 1.
- 2) Schedule a meeting to discuss how industry will address AI 7 once the draft SE for BWRVIP-234 has been issued.

For AI 7, the NRC staff reported that the draft safety evaluation (SE) for Boiling Water Reactor Vessel Internals Project (BWRVIP)-234, "Thermal Aging and Neutron Embrittlement Evaluation of Cast Austenitic Stainless Steel for BWR Internals," would be complete and issued mid to late January 2016. BWRVIP-234 contains screening criteria that are applicable to AI 7. It was agreed that once the draft SE was issued, a meeting be held in late January to discuss how industry would address AI 7.

Actions from the meeting were:

- 1) Schedule a meeting in spring 2016 to discuss options and recommendations to address AI 1.
- 2) Schedule a meeting to discuss how industry will address AI 7 once the draft SE for BWRVIP-234 has been issued.

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