



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 2, 2016

Mr. David A. Heacock
President and Chief Nuclear Officer
Virginia Electric and Power Company
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: NORTH ANNA POWER STATION, UNITS 1 AND 2 (NAPS) - ISSUANCE OF AMENDMENTS FOR CHANGES TO THE EMERGENCY ACTION LEVEL SCHEME - CORRECTION OF EQUIPMENT IDENTIFICATION NUMBER (CAC NOS. MF6220 AND MF6221)

Dear Mr. Heacock:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 277 and 259 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station (NAPS), Units 1 and 2, respectively. These amendments are in response to your application dated May 4, 2015, and as supplemented August 5, 2015, for the proposed changes to the Emergency Action Level (EAL).

The amendments authorize modification of the NAPS EAL Technical Basis Document, EAL RA2.1, to revise the EAL classification scheme because of a replaced instrument which uses a different alarm scale for comparable radiological measurements. Specifically, these amendments change the equipment identification number for the "GW-RI-178-1 Process Vent Normal Range" monitor to "VG-RI-180-1 Vent Stack 'B' Normal Range" for Initiating Condition RA2, EAL RA2.1.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "V. Sreenivas", with a stylized flourish at the end.

Dr. V. Sreenivas, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosures:

1. Amendment No. 277 to NPF-4
2. Amendment No. 259 to NPF-7
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 277
Renewed License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated May 4, 2015, as supplemented by letter dated August 5, 2015, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 1

2. Accordingly, the license is amended by changes to paragraph 2.C (2) of Renewed Facility Operating License No. NPF-4, as indicated in the attachment to this license amendment, and is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 277, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. Further Renewed Facility Operating License No. NPF-4 is hereby amended to authorize changes to the North Anna Emergency Action Level (EAL) scheme to revise EAL RA2.1 from "GW-RI-178-1 Process Vent Normal Range" to "VG-RI-180-1 Vent Stack 'B' Normal Range" and to clarify the alarm setpoint nomenclature associated with VG-RI-180-1 (i.e., change "Hi-Hi" to "high"). Although the changes to the NAPS EAL Technical Bases Document will revise the instrumentation used to classify an event under this EAL with instrumentation that utilizes a different alarm scale for comparable radiological measurements, the underlying guidance and basis for the EAL (plant-specific indication of fuel uncover or damage) remains unchanged.
4. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



William M. Dean, Director
Office of Nuclear Reactor Regulation

Attachment:
Changes to License No. NPF-4

Date of Issuance: February 2, 2016



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 259
Renewed License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated May 4, 2015, as supplemented by letter dated August 5, 2015, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 2

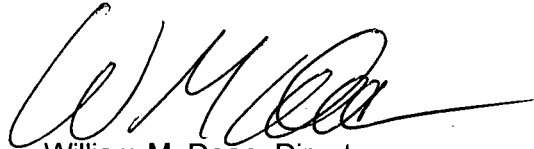
2. Accordingly, the license is amended by changes to paragraph 2.C (2) of Renewed Facility Operating License No. NPF-7, as indicated in the attachment to this license amendment, and is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 259, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. Further Renewed Facility Operating License No. NPF-7 is hereby amended to authorize changes to the North Anna Emergency Action Level (EAL) scheme to revise EAL RA2.1 from "GW-RI-178-1 Process Vent Normal Range" to "VG-RI-180-1 Vent Stack 'B' Normal Range" and to clarify the alarm setpoint nomenclature associated with VG-RI-180-1 (i.e., change "Hi-Hi" to "high"). Although the changes to the NAPS EAL Technical Bases Document will revise the instrumentation used to classify an event under this EAL with instrumentation that utilizes a different alarm scale for comparable radiological measurements, the underlying guidance and basis for the EAL (plant-specific indication of fuel uncover or damage) remains unchanged.
4. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



William M. Dean, Director
Office of Nuclear Reactor Regulation

Attachment:
Changes to License No. NPF-7

Date of Issuance: February 2, 2016

ATTACHMENT TO
LICENSE AMENDMENT NO. 277
RENEWED FACILITY OPERATING LICENSE NO. NPF-4
DOCKET NO. 50-338
AND LICENSE AMENDMENT NO. 259
RENEWED FACILITY OPERATING LICENSE NO. NPF-7
DOCKET NO. 50-339

Replace the following pages of the Licenses with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

License Pages

License No. NPF-4, page 3
License No. NPF-7, page 3

Insert

License Pages

License No. NPF-4, page 3
License No. NPF-7, page 3

- (2) Pursuant to the Act and 10 CFR Part 70, VEPCO to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level
VEPCO is authorized to operate the North Anna Power Station, Unit No. 1, at reactor core power levels not in excess of 2940 megawatts (thermal).
 - (2) Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 277 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations as set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

VEPCO is authorized to operate the facility at steady state reactor core power levels not in excess of 2940 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 259 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the condition or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission:

- a. If VEPCO plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the North Anna Power Station, the



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO EMERGENCY ACTION LEVEL SCHEME CHANGES

AMENDMENT NO. 277 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-4,

AND

AMENDMENT NO. 259 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-7,

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-338 AND 50-339

1.0 INTRODUCTION

By application dated May 4, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15131A026), and supplemented by letter dated August 5, 2015 (ADAMS Accession No. ML15224A547), Virginia Electric and Power Company (Dominion, the licensee) submitted changes in accordance with Sections 50.90 and 50.54(q) of Title 10 of the *Code of Federal Regulations* (10 CFR) to the Emergency Action Level (EAL) Technical Basis Document for the North Anna Power Station (NAPS), Units 1 and 2. The proposed changes would modify the NAPS EAL Technical Basis Document, EAL RA2.1, to revise the instrumentation used to classify an event under this EAL. This instrumentation was replaced with instrumentation that utilizes a different equipment identification number and alarm scale. These changes were submitted under 10 CFR 50.54(q) identified by the licensee as a reduction in effectiveness to the NAPS EAL Technical Bases Document, and as such, the licensee has requested NRC approval prior to implementation.

The proposed change would replace "GW-RI-178-1 Process Vent Normal Range" monitor as an available instrument to assess the EAL with "VG-RI-180-1 Vent Stack 'B' Normal Range" monitor. It would also clarify the alarm setpoint nomenclature associated with the VG-RI-180-1 instrument design to reflect comparable radiological measurements (i.e., change "Hi-Hi" to "high").

The supplemental letter dated August 5, 2015, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on July 7, 2015 (80 FR 38764).

Enclosure 3

2.0 REGULATORY EVALUATION

The applicable regulations and guidance for the emergency plans are as follows:

2.1 Regulations

Section 50.47(b) of 10 CFR establishes the standards that the onsite and offsite emergency response plans must meet for NRC staff to make a positive finding that there is reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency.

Planning Standard (4) of this section requires that onsite and offsite emergency response plans contain:

A standard emergency classification and action level scheme, the bases of which include facility system and effluent parameters, is in use by the nuclear facility licensee, and State and local response plans call for reliance on information provided by facility licensees for determinations of minimum initial offsite response measures.

Section IV.B of Appendix E, "Emergency Planning and Preparedness for Production and Utilization Facilities," to 10 CFR Part 50, states, in part:

The means to be used for determining the magnitude of, and for continually assessing the impact of, the release of radioactive materials shall be described, including emergency action levels that are to be used as criteria for determining the need for notification and participation of local and State agencies, the Commission, and other Federal agencies, and the emergency action levels that are to be used for determining when and what type of protective measures should be considered within and outside the site boundary to protect health and safety. The emergency action levels shall be based on in-plant conditions and instrumentation in addition to onsite and offsite monitoring. By June 20, 2012, for nuclear power reactor licensees, these action levels must include hostile action that may adversely affect the nuclear power plant.

2.2 Guidance

The licensee's current NRC-approved EAL scheme is based on Nuclear Energy Institute (NEI) 99-01, "Methodology for Development of Emergency Action Levels," Revision 4 (ADAMS Accession No. ML041470143). This NRC-endorsed guidance provides licensees with an acceptable method to develop and implement a site-specific EAL scheme to meet the requirements of Section IV of Appendix E to 10 CFR Part 50 and 10 CFR 50.47(b)(4).

NRC Regulatory Issue Summary (RIS) 2003-18, with Supplements 1 and 2, "Use of NEI 99-01, Methodology for Development of Emergency Action Levels," (ADAMS Accession Nos. ML032580518, ML041550395, ML051450482) provides guidance for developing or changing a standard emergency classification and action level scheme. In addition, this RIS and its supplements provide recommendations to assist licensees, consistent with Section IV.B of

Appendix E to Part 50, in determining whether to seek prior NRC approval of deviations from the guidance.

3.0 TECHNICAL EVALUATION

3.1 Background

By letter dated February 4, 2008 (ADAMS Accession No. ML080310195), the NRC approved an EAL scheme change for the NAPS to one based on NEI 99-01, "Methodology for Development of Emergency Action Levels," Revision 4. This scheme includes EAL RA2.1, based upon plant-specific indication of fuel uncover or damage, including spent fuel stored in the spent fuel pool or refueling pathway.

During a recent emergency exercise, the licensee identified an administrative error in the current EAL RA2.1. This EAL listed an incorrect instrument as the indicator of a potential release path. "GW-RI-178-1 Process Vent Normal Range" was identified as the incorrect instrument listed in the EAL because that instrument did not monitor damage to fuel outside of the reactor vessel, as required by EAL RA2.1. An Operations Department Standing Order associated with this EAL was issued as an operator aid to address the administrative error until this EAL was revised. The proposed change would replace "GW-RI-178-1 Process Vent Normal Range" monitor as an available instrument to assess the EAL with "VG-RI-180-1 Vent Stack 'B' Normal Range" monitor. "VG-RI-180-1 Vent Stack 'B' Normal Range" was identified as the correct instrument to be listed in EAL RA2.1 because it monitors ventilation systems outside of the reactor vessel, as required by EAL RA2.1.

Using the guidance contained in 10 CFR 50.54(q), the licensee determined that this change constituted a reduction in effectiveness to the NAPS EAL Technical Basis Document, requiring NRC approval prior to implementation. The licensee determined that this change constituted a reduction in effectiveness because the EAL will be changed such that classification of an event could be at a different threshold from the EAL approved in the NRC's 2008 Safety Evaluation Report (SER). Furthermore, the licensee's supplement requested that the amendment alarm setpoint nomenclature be changed to reflect the differing instrument design to reflect comparable radiological measurements between the two monitors from different manufacturers. This was requested to provide clarification for the operators by identifying the correct alarm setpoint nomenclature.

3.2 NRC Staff Evaluation

In considering the proposed changes to the NAPS EAL Technical Basis Document, the NRC staff considered:

1. The instrumentation and setpoints derived for the EAL and the impact to a standard EAL scheme, as required by 10 CFR 50.47(b)(4);
2. The plant-specific implementation method for the EAL and the impact to the licensee's ability to meet the requirements of Section IV of Appendix E to 10 CFR Part 50; and

3. The basis for the EAL and its consistent application to the NRC-approved EAL scheme for NAPS.

EAL RA2.1 is based upon plant-specific indication of fuel uncover or damage, including spent fuel stored in the spent fuel pool or refueling pathway.

The proposed changes will replace "Process Vent Normal Range Monitor GW-RM-178-1" with "Vent Stack 'B' Normal Range Monitor VG-RI-180-1" in the EAL scheme. The licensee stated that the purpose of "Process Vent Normal Range Monitor GW-RM-178-1" was to provide monitoring for certain vents and tanks outside the reactor vessel. Also, one purpose of the "Vent Stack 'B' Normal Range Monitor VG-RI-180-1" is to provide ventilation radiation monitoring for the Fuel Building, and as such, this radiation monitor would be capable of monitoring for damage to fuel outside of the reactor vessel. These amendments revise the EAL Technical Basis document to properly reflect the monitoring instruments actually in place to comply with EAL RA2.1 and to accurately classify an event under the EAL scheme, as required by 50.47(b)(4) and Section IV of Appendix E to Part 50. The change in the alarm setpoint reflects the differences in nomenclature used between the two monitors' different manufacturers for comparable detection ranges and provides clarification to the operators regarding the correct equipment identification number and allows operators to detect and classify an event properly under the EAL scheme.

The licensee sought NRC approval for this change based on its determination under 10 CFR § 50.54(q) that this amendment would result in a reduction in effectiveness of the EAL Technical Basis document. Although the proposed changes to the NAPS EAL Technical Basis Document would revise the instrumentation used to classify an event under this EAL with instrumentation that utilizes a different equipment identification number and noun name, the underlying guidance and basis for the EAL (plant-specific indication of fuel uncover or damage) remains unchanged.

The instrumentation and setpoints derived for this EAL are consistent with the overall EAL scheme development guidance, address the plant-specific implementation strategies provided, and are consistent with a standard EAL scheme, as required by 10 CFR 50.47(b)(4). Therefore, the incorporation of corrected instrumentation for this EAL, with the appropriate setpoint to indicate fuel uncover or damage, does not alter the intent, implementation method, or progression through the EAL scheme.

Based on the above, the NRC staff has determined that the proposed changes meet the requirements in Appendix E to 10 CFR Part 50 and the planning standards of 10 CFR 50.47(b). Therefore, the NRC staff concludes that the licensee's proposed EAL change, as proposed in their letter dated May 4, 2015, and as supplemented by a letter dated August 5, 2015, is acceptable and provides reasonable assurance that the licensee can and will take adequate protective measures in the event of a radiological emergency. Therefore, the NRC staff concludes that the plant-specific implementation method for this EAL set is in alignment with the key characteristics of an effective EAL scheme, meets the requirements of Section IV of Appendix E to 10 CFR Part 50 and 10 CFR 50.47(b)(4), and is, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, officials from the Commonwealth of Virginia were notified of the proposed issuance of the amendment. The NRC did not receive any comments from the Commonwealth officials.

5.0 PUBLIC COMMENTS

On July 7, 2015, the NRC staff published a "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing," in the *Federal Register* associated with the proposed amendment request (80 FR 38764). In accordance with the requirements in 10 CFR 50.91, "Notice for public comment; State consultation," the notice provided a 30-day period for public comment on the proposed no significant hazards consideration (NSHC) determination. The 30-day comment period ended on August 6, 2015. Two public comments were received regarding the proposed amendment. These comments were subject to the Virginia earthquake in 2011, leak-before-break procedures, and cyber attacks. The NRC staff review concluded that these comments were not related to the amendment request regarding the proposed EAL change and did not affect the conclusions in the NSHC determination as published in the *Federal Register*. Therefore, no changes were made to EAL approval as described in this safety evaluation.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. This amendment does not alter any operational, maintenance, or substantive surveillance requirements, but only changes which monitor is identified and how it is to be read to better conform to the EAL scheme. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration (80 FR 38764). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: O. Aragon, NSIR
Don Johnson, NSIR

Date of issuance: February 2, 2016

February 2, 2016

Mr. David A. Heacock
President and Chief Nuclear Officer
Virginia Electric and Power Company
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: NORTH ANNA POWER STATION, UNITS 1 AND 2 (NAPS) - ISSUANCE OF AMENDMENTS FOR CHANGES TO THE EMERGENCY ACTION LEVEL SCHEME – CORRECTION OF EQUIPMENT IDENTIFICATION NUMBER (CAC NOS. MF6220 AND MF6221)

Dear Mr. Heacock:

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A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,
/RA/
Dr. V. Sreenivas, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosures:

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cc: Listserv

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*SE input

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OFFICE	LPL2-1/BC	DORL/D	NRR/D	LPL2-1/PM	
NAME	MMarkley	ABoland (PKrohn for)	WDean	VSreenivas	
DATE	01/21/16	01/27/16	02/02/16	02/02/16	

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