



Dresden Generating Station

6500 North Dresden Road

Morris, IL 60450

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10 CFR 50.73

SVPLTR # 15-0060

October 29, 2015

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Dresden Nuclear Power Station, Unit 3
Renewed Facility Operating License No. DPR-25
NRC Docket No. 50-249

Subject: Licensee Event Report 249/2015-001-00, Main Steam Line Flow Switches Found Outside Tech Spec Allowed Value

Enclosed is Licensee Event Report 249/2015-001-00, "Main Steam Line Flow Switches Found Outside Tech Spec Allowed Value." This is an interim report which describes events which are being reported in accordance with 10 CFR 50.73(a)(2)(i)(B), "any operation or condition which was prohibited by the plant's Technical Specifications".

There are no regulatory commitments contained in this submittal.

Should you have any questions concerning this letter, please contact Mr. Bruce Franzen at (815) 416-2800.

Respectfully,

A handwritten signature in black ink that reads "Shane Marik".

Shane M. Marik
Site Vice President
Dresden Nuclear Power Station

Enclosure Licensee Event Report 249/2015-001-00

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Dresden Nuclear Power Station

IE22
NRR



LICENSEE EVENT REPORT (LER)
(See Page 2 for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Dresden Nuclear Power Station, Unit 3	2. DOCKET NUMBER 05000249	3. PAGE 1 OF 3
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4. TITLE
Main Steam Line Flow Switches Found Outside Tech Spec Allowed Value

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	05	2015	2015	001	00	10	29	15	FACILITY NAME	DOCKET NUMBER

9. OPERATING MODE **11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)**

1	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
10. POWER LEVEL 100	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

LICENSEE CONTACT Bruce Franzen – Regulatory Assurance Manager	TELEPHONE NUMBER (Include Area Code) 815-416-2800
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
X	SB	FIS	I024	Y					

14. SUPPLEMENTAL REPORT EXPECTED **15. EXPECTED SUBMISSION DATE**

YES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO

MONTH	DAY	YEAR
01	18	16

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

During quarterly calibration and functional testing of Main Steam (SB) Line High Flow switches, two switches were found to not meet surveillance requirements of Technical Specification 3.3.6.1.

At 1100 CDT on September 5, 2015, a switch was found to not meet Surveillance Requirement (SR) 3.3.6.1.1. TS 3.3.6.1 Condition A and D completion times were not met and TS 3.0.2 was entered. Maintenance personnel repaired the affected instrument. No loss of safety function occurred due to this failure.

These events are being reported under 10 CFR 50.73(a)(2)(i)(B) "Any operation or condition which was prohibited by the plant's Technical Specifications...". No loss of safety function occurred due to these events.

The cause of these issues is under investigation. Immediate corrective actions include replacement of the failed components and remediation of the team which failed to identify the Tech Spec condition. These events are considered of very low safety significance.



**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

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NARRATIVE

PLANT AND SYSTEM IDENTIFICATION

Dresden Nuclear Power Station (DNPS), Unit 3, is a General Electric Company Boiling Water Reactor with a licensed maximum power level of 2957 megawatts thermal. The Energy Industry Identification System codes used in the text are identified as [XX].

A. Plant Conditions Prior to Event:

Unit: 03 Event Date: 09/05/15 to 09/11/15 Event Time: 1100 CDT
 Reactor Mode: 1 Mode Name: Power Operation Power Level: 100 percent

B. Description of Event:

During quarterly calibration and functional testing of Main Steam [SB] Line High Flow switches, two switches were found to not meet surveillance requirements of Technical Specification 3.3.6.1. This event is being reported under 10 CFR 50.73(a)(2)(i)(B) "Operation or Condition Prohibited by Technical Specifications" due to multiple test failures of similar components.

Additionally, at approximately 1100 CDT on September 5, 2015, Operations logged the failure of a switch to meet SR 3.3.6.1.1 channel check criteria. September 7, 2015, TS 3.3.6.1 Condition A and D completion times had not been met and Tech Spec 3.0.2 was entered. Maintenance personnel repaired the affected instrument. This event is being reported under 10 CFR 50.73(a)(2)(i)(B) "Any operation or condition which was prohibited by the plant's Technical Specifications...".

No individual failure caused a loss of safety function. The aggregate impact of these events did not result in the loss of safety function.

C. Cause of Event:

The cause of these issues is under investigation.

**LICENSEE EVENT REPORT (LER)
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NARRATIVE

D. Safety Analysis:

The automatic isolation closure of the main steam line isolation valves was maintained due to system redundancy. No loss of safety function occurred, therefore these failures are considered of very low safety significance. Further safety analysis will be completed by the investigation.

E. Corrective Actions:

Immediate corrective actions included replacement of the failed components and remediation of the Operations team which failed to identify the Tech Spec condition. Further corrective actions will be determined through the open investigation.

F. Previous Occurrences:

A review of the previous 12 quarterly surveillances, three years, was performed. No previous occurrences were identified. Further occurrences will be determined through the corrective action program.

G. Component Failure Data:

Manufacturer	Model	S/N	Type
ITT Barton	288A and 289A	TBD	Meter Movement for Differential pressure switch