Nuclear Operating Company

South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

September 30, 2015
NOC-AE-15003295
10CFR50.59
STI: 34200838

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

South Texas Project
Units 1 & 2
Docket Nos. STN 50-498, STN 50-499
10CFR50.59 Summary Report

Pursuant to the requirements of 10CFR50.59, the attached report contains a brief description and summary of the 10CFR50.59 evaluations of changes, tests and experiments conducted at the South Texas Project from October 2013 through September 2015.

There are no commitments in this letter.

If there are any questions regarding this summary report, please contact Hung C. Le at (361) 972-7932 or me at (361) 972-8164.

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hcl

Attachment: 10CFR50.59 Evaluation Summaries October 2013 – September 2015

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NRR

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10CFR50.59 Evaluation Summaries
October 2013 – September 2015

Summaries of the following 10CFR50.59 Evaluations are provided in this attachment:

No.	Condition Report Number	Subject
1.	11-30675-90	Methodology change from the PHOENIX-P computer code to the NEXUS/PARAGON computer codes.
2.	04-8245-60	High radiation monitor equipment failure setpoint change
3.	11-12472-20	UFSAR revision due to revised Reactor Containment Building Loss Of Coolant Accident Mass and Energy

10CFR50.59 Evaluation Summaries October 2013 – September 2015

1. **11-30675-90 - Methodology Change from the PHOENIX-P computer code to the NEXUS/PARAGON computer codes.**

   Description: The subject of this evaluation is to revise the methodology in the UFSAR for the development of cross sections used by the ANC computer code from the PHOENIX-P computer code to the NEXUS/Paragon computer codes.

   Summary: These computer codes have been reviewed and approved for use by the NRC. The evaluation demonstrated that the proposed change uses a NRC approved methodology that is (a) based on sound engineering practice, (b) appropriate for the intended application, and (c) within the limitations of the applicable Safety Evaluation Reports.
   The evaluation determined that prior NRC approval was not required.

2. **04-8245-60 - High Radiation Monitor Equipment Failure Setpoint Change**

   Description: The subject of this evaluation is to revise the equipment failure setpoint for the High Range Radiation Monitors (HRRM's) from 5 minutes to 30 minutes.

   Summary: The change is being made to eliminate a false indication of a common mode failure of the HRRM's during a high energy line break.
   The evaluation determined that prior NRC approval was not required.

3. **11-12472-20 - UFSAR Revision due to Revised Reactor Containment Building Loss Of Coolant Accident Mass and Energy**

   Description: The proposed change revises UFSAR Section 6.2.1 to reflect revised containment atmospheric pressure and temperature analysis for the LOCA event.

   Summary: The analysis was re-performed due to revised mass and energy releases to the containment. The GOTHIC computer code was used in place of the currently licensed CONTEMPT computer code. The results of the analysis show that the peak containment pressure of 41.2 psig in Technical Specification 6.8.3.j remains unchanged. The results also show that the peak containment temperature decreases to 262°F (GOTHIC) from 264°F (CONTEMPT) and remains bounded by the Main Steam Line Break peak temperature of 299°F.
   The evaluation determined that prior NRC approval was not required.