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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

November 23, 2015

Mr. Bryan C. Hanson  
President and Chief Nuclear Officer  
Exelon Nuclear  
Nine Mile Point Nuclear Station, LLC  
4300 Winfield Road  
Warrenville, IL 60555

SUBJECT: NINE MILE POINT NUCLEAR STATION, UNIT NO. 2 - CORRECTION LETTER  
TO AMENDMENT NO. 151 RE: MAXIMUM EXTENDED LOAD LINE LIMIT  
ANALYSIS PLUS (CAC NO. MF3056)

Dear Mr. Hanson:

By letter dated September 2, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15230A487), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 151, "Maximum Extended Load Line Limit Analysis Plus" (MELLLA+), to Renewed Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit No. 2 (NMP2). The amendment consists of changes to the technical specifications (TSs) in response to your application transmitted by letter dated November 1, 2013, as supplemented by letters dated January 21, February 14, February 25, March 10, May 14, June 13, October 10, December 11, 2014, and February 18, 2015.

By emails dated September 29 and October 21, 2015 (non-publicly available), and the discussion with your staff on October 21, 2015, they identified: (1) a correction to TS page 3.3.1.1-9, and (2) several instances where the publicly-available MELLLA+ safety evaluation (SE) (ADAMS Accession No. ML15223B144) contained sensitive proprietary information that has been submitted previously by the licensee and approved by the NRC for withholding from public disclosure in accordance with Section 2.390 of Title 10 of the *Code of Federal Regulations* (10 CFR). The NRC acknowledges the error on the TS page, as well as associated portions of the NRC SE that should have been withheld from public disclosure in accordance with Section 2.390 of 10 CFR, and has replaced the publicly-available version of the MELLLA+ SE, in its entirety, in ADAMS under the existing Accession No. ML15223B144.

NOTICE: Enclosure 3 to this letter contains Sensitive Proprietary Information  
Upon separation from the Enclosure 3, this letter is DECONTROLLED

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B. Hanson

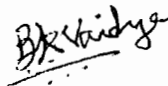
- 2 -

The corrected TS page (TS 3.3.1.1-9) and corrected publicly-available SE that reflects the additional redacted information is enclosed. Also, for consistency, the non-publicly available version of the MELLA+ SE has been updated and replaced in ADAMS under the existing Accession No. ML15195A257.

These corrections do not change the amendment pages of the license or any of the conclusions in the SE associated with the amendment, and do not affect the associated notice to the public.

Please contact me at 301-415-3308 if you have any questions.

Sincerely,



Bhalchandra K. Vaidya, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-410

Enclosure:

- 1) Corrected TS Page TS 3.3.1.1-9
- 2) Corrected Non-proprietary Safety Evaluation (ML15223B144)
- 3) Corrected Proprietary Safety Evaluation

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Table 3.3.1.1-1 (page 2 of 3)  
Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION D.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. Average Power Range Monitors (continued)					
e. OPRM-Upscale	≥18% RTP(f)	3 per logic channel	F	SR 3.3.1.1.2 SR 3.3.1.1.7 SR 3.3.1.1.10 SR 3.3.1.1.13	NA
f. 2-Out-Of-4 Voter	1,2	2	H	SR 3.3.1.1.2 SR 3.3.1.1.10 SR 3.3.1.1.14 SR 3.3.1.1.17	NA
3. Reactor Vessel Steam Dome Pressure – High	1,2	2	H	SR 3.3.1.1.1 SR 3.3.1.1.8 SR 3.3.1.1.9 SR 3.3.1.1.13 SR 3.3.1.1.14 SR 3.3.1.1.17	≤ 1072 psig
4. Reactor Vessel Water Level – Low, Level 3	1,2	2	H	SR 3.3.1.1.1 SR 3.3.1.1.8 SR 3.3.1.1.9 SR 3.3.1.1.13 SR 3.3.1.1.14 SR 3.3.1.1.17	≥ 157.8 inches
5. Main Steam Isolation Valve – Closure	1	8	G	SR 3.3.1.1.8 SR 3.3.1.1.13 SR 3.3.1.1.14 SR 3.3.1.1.17	≤ 12% closed
6. Drywell Pressure – High	1,2	2	H	SR 3.3.1.1.1 SR 3.3.1.1.8 SR 3.3.1.1.9 SR 3.3.1.1.13 SR 3.3.1.1.14	≤ 1.88 psig

(continued)

- (f) Following DSS-CD implementation, DSS-CD is not required to be armed while in the DSS-CD Armed Region during the first reactor startup and during the first controlled shutdown that passes completely through the DSS-CD Armed Region. However, DSS-CD is considered OPERABLE and shall be maintained OPERABLE and capable of automatically arming for operation at recirculation drive flow rates above the DSS-CD Armed Region.

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These corrections do not change the amendment pages of the license or any of the conclusions in the SE associated with the amendment, and do not affect the associated notice to the public.

Please contact me at 301-415-3308 if you have any questions.

Sincerely,  
*/RA/*  
Bhalchandra K. Vaidya, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-410

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**ADAMS Accession Nos.: Package: ML15299A049**  
**Correction Letter and corrected TS Page 3.3.1.1-9: ML15296A056**  
**Email from Licensee identifying the inadvertent release of Proprietary information: ML15295A018**  
**Corrected Non-Proprietary SE: ML15223B144      Corrected Proprietary SE: ML15195A257**

OFFICE	NRR/DORL/LPLIII-2/PM	NRR/DORL/LPLI-1/LA	NRR/DORL/LPLI-1/BC	NRR/DORL/LPLIII-2/PM
NAME	BVaidya	KGoldstein (MHenderson for)	TTate	BVaidya
DATE	11/18/2015	11/18/2015	11/23/2015	11/23/2015

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